

OFFICE OF CIVILIAN RADIOACTIVE WASTE MANAGEMENT
SPECIAL INSTRUCTION SHEET

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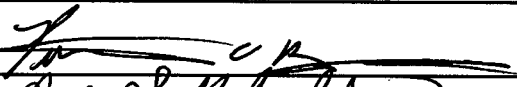

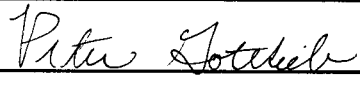
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1. PURPOSE

The Monitored Geologic Repository (MGR) Waste Package Department of the Civilian Radioactive Waste Management System Management & Operating Contractor (CRWMS M&O) performed calculations to provide input for disposal of spent nuclear fuel (SNF) from the N Reactor, a graphite moderated reactor at the Department of Energy's (DOE) Hanford Site (Ref. 1). The N Reactor core was fueled with slightly enriched (0.947 wt% and 0.947 to 1.25 wt% ^{235}U in Mark IV and Mark IA fuels, respectively) U-metal clad in Zircaloy-2 (Ref. 1, Sec. 3). Both types of N Reactor SNF have been considered for disposal at the proposed Yucca Mountain site. For some WPs, the outer shell and inner shell may breach (Ref. 3) allowing the influx of water. Water in the WP will moderate neutrons, increasing the likelihood of a criticality event within the WP; and the water may, in time, gradually leach the fissile components from the WP, further affecting the neutronics of the system.

This study presents calculations of the long-term geochemical behavior of WPs containing two multi-canister overpacks (MCO) with either six baskets of Mark IA or five baskets of Mark IV intact N Reactor SNF rods (Ref. 1, Sec. 4) and two high-level waste (HLW) glass pour canisters (GPCs) arranged according to the codisposal concept (Ref. 4). The specific study objectives were to determine:

1. The extent to which fissile uranium will remain in the WP after corrosion/dissolution of the initial WP configuration
2. The extent to which fissile uranium will be carried out of the degraded WP by infiltrating water (such that internal criticality is no longer possible, but the possibility of external criticality may be enhanced)
3. The nominal chemical composition for the criticality evaluations of the WP design, and to suggest the range of parametric variations for additional evaluations.

The scope of this calculation, the chemical compositions (and subsequent criticality evaluations) of the simulations, is limited to time periods up to 6.35×10^5 years. This longer time frame is closer to the one million year time horizon recently recommended by the National Academy of Sciences to the Environmental Protection Agency for performance assessment related to a nuclear repository (Ref. 5). However, it is important to note that after 100,000 years, most of the materials of interest (fissile materials) will have either been removed from the WP, reached a steady state, or been transmuted.

This document has been prepared according to Administrative Procedure AP-3.12Q (Ref. 37), *Calculations*, and is subject to the Quality Assurance Requirements and Description (QARD) Document (Ref. 41) requirements. This calculation has been prepared in accordance with the *Technical Work Plan for: Department of Energy Spent Nuclear Fuel Work Packages* (Ref. 8).

2. METHOD

The method used for this analysis involves the following steps:

- Use of basic EQ3/6 (software package, described in Section 4.1 and in Ref. 22) capability for tracing the progress of reactions with evolution of the chemistry, which includes the estimation of the concentrations remaining in solution and the composition of the precipitated solids. (EQ3 is used to determine a starting fluid composition for EQ6 calculations; it does not simulate reaction progress.)
- Evaluation of available data on the range of dissolution rates for the materials involved, to be used as material/species input for each time step.
- Use of "solid-centered flow-through" mode (SCFT) in EQ6; in this mode, an increment of aqueous "feed" solution is added continuously to the WP system, and a like volume of the existing solution is removed, simulating a continuously-stirred tank reactor. This mode is discussed in Section 4.
- Determination of fissile material concentrations in solution as a function of time (from the output of EQ6 simulated reaction times up to 6.35×10^5 years).
- Calculation of the amount of fissile material released from the WP as a function of time (fissile material loss reduces the chance of criticality within the WP).
- Composition and amounts of solids (precipitated minerals or corrosion products, and unreacted WP materials).

This calculation used version 7.2bLV (Ref. 12) of EQ6 (Refs. 22, 23, 24, and 25). The User's Manual for 7.2bLV (Ref. 13) provides a detailed description of the code. The code retains the solid-centered flow-through (SCFT) mode developed in the previous Addendum to EQ6 (Ref. 18) and is complete, mathematically correct and technically adequate for the application. Further detail on the specific methods employed for each step is available in Section 5 of this calculation.

With regard to the development of this calculation, the control of the electronic management of data was evaluated in accordance with AP-SV.1Q, *Control of the Electronic Management of Information* (Ref. 15). The evaluation (Ref. 8) determined that current work processes and procedures are adequate for the control of electronic management of data for this activity.

3. ASSUMPTIONS

All assumptions are for preliminary design. All assumptions are used throughout Section 5.

- 3.1 The Enhanced Design Alternative (EDA) II waste package (Ref. 26) is assumed for disposal of N Reactor SNF. The rationale for this assumption is that the EDA II design superseded the Viability Assessment (VA) design before the N Reactor calculations began.
- 3.2 It is assumed that the solutions that drip into the WP will have the major ion composition of J-13 well water as given in Ref. 54 (Document Tracking Number (DTN): MO0006J13WTRCM.000), and that minor components in the solution can be approximated by Reference 6 for 6.35×10^5 years. The rationale for this assumption is that the groundwater composition is controlled largely by transport through the host rock, over pathways of hundreds of meters, and the host rock composition is not expected to change substantially over 10^6 years. The assumption that the water entering the WP can be approximated by the J-13 well water implicitly assumes that any effects of contact with the engineered materials in the drift will be minimal after a few thousand years. For a few thousand years after waste emplacement, the composition may differ because of perturbations resulting from reactions with engineered materials and from the thermal pulse. These are not taken into account in this calculation because the outer shell and inner liner are not expected to breach until after that perturbed period. Therefore, the early perturbation is not relevant to the calculations reported in this document.
- 3.3 It is assumed that the density of J-13 well water is 1.0 g/cm^3 . The rationale of this assumption is that in dilute solutions, the density is extremely close to that for pure water and that any differences are insignificant with respect to other uncertainties in the data and calculations. Moreover, this number is used only initially in EQ3/6 to convert concentrations of dissolved substances from parts per million to molality.
- 3.4 It is assumed that an aqueous solution fills all voids within the waste package. The rationale for this assumption is that it provides the maximum degradation rates of WP components with the potential for precipitation of radionuclides within the WP or the flushing of radionuclides from the WP, and is therefore conservative. This assumption is justified by recent evaluations of codisposal WPs (Ref. 2) which show that degradation of the WP materials (specifically, HLW glass and steel) overwhelms the native chemistry of the incoming water (Figures 5-2 through 5-20 of Ref. 2 show pH variations of 3 to 10 in WP)
- 3.5 It is assumed that water will circulate freely enough in the partially degraded WP that all degraded solid products will react with each other through the aqueous solution medium. The rationale for this assumption is that it provides the most rapid aqueous degradation and is, therefore, conservative with respect to criticality.

- 3.6 It is assumed that data in the 25°C thermodynamic database can be used for the calculation. The rationale for this assumption is that though the initial breach of the WP may occur when the WP contents are at temperatures $\geq 50^{\circ}\text{C}$ (Ref. 7, Figures 3-20 through 3-22), at times $> 25,000$ years, the WP temperatures are likely to be close to 25°C. It is further assumed that the thermodynamic database used for these calculations (data0.ymp.ROA) does not differ from the database recently qualified for project use (data0.ymp.R0) in any way that would cause a significant impact to the calculations reported in this report. The rationale for this is that the changes made in the qualification process included removal of data extrapolated to elevated temperatures, which are not of concern to this study, and the correction of numerical errors due to transcription of data from the source references. Future revisions to this document will use the qualified database, and the impact of changes can be assessed at that time.
- 3.7 In general it is assumed that chromium and molybdenum will oxidize fully to chromate (or dichromate) and molybdate, respectively. This assumption is based on the available thermodynamic data, which indicate that in the presence of air, the chromium and molybdenum would both oxidize to the VI valence state. Laboratory observation of the corrosion of Cr and Mo containing steels and alloys, however, indicates that any such oxidation would be extremely slow. In fact, oxidation to the VI state may not occur at a significant rate with respect to the time frame of interest, or there may exist stable Cr(III) solids that substantially lower aqueous Cr concentration. For the present analyses, the assumption is made that, over the times of concern, oxidation will occur. The rationale for this assumption is that by allowing the Cr and Mo to oxidize, the pH of the system will be lowered allowing for the removal of neutron absorbers. This is conservative for internal criticality since most of the Pu and U would remain in the waste package.
- 3.8 The composition of the aqueous solution that will enter the waste package will not be altered by contact with the material. The rationale for this assumption is that the water is expected to move rapidly enough through openings in the waste package materials that its residence time in the corroded barrier will be too small for significant reaction to occur. Furthermore, the water flowing through the barriers will be in contact with the corrosion products left from the barrier corrosion that created the holes initially. These corrosion products will closely resemble iron oxides and hydroxides in the overlying rock. Consequently, the water should already be close to equilibrium with these compounds and would be unaffected by further contact with them, even if flow slowed enough to permit significant reaction.
- 3.9 It is assumed that gases in the WP solution remain in equilibrium with the ambient atmosphere outside the WP. In other words, contact of WP fluids with the gas phase in the repository is envisioned to be sufficient to maintain equilibrium with the CO_2 and O_2 present, whether or not this is the normal atmosphere in open air or rock gas that seeps out of the adjacent tuff. Moreover, the specific partial pressures of CO_2 and O_2 of the ambient repository atmosphere are set to, respectively, $10^{-3.0}$ and $10^{-0.7}$ atm. The rationale

for the oxygen partial pressure is that it is equivalent to that in the atmosphere (Ref. 10, p. F-210). The rationale for choosing the carbon dioxide pressure is to reflect the observation that J-13 well water appears to be in equilibrium with above-atmospheric carbon dioxide levels (Ref. 9, Table 7;).

- 3.10 It is assumed that precipitated solids are deposited, remain in place, and are not mechanically eroded or entrained as colloids in the advected water. The rationale for this assumption is that since dissolved fissile material (U, Pu, Np) may be adsorbed on colloids (clays, iron oxides) or may be precipitated as colloids during WP degradation (Ref. 7, Sec. 3.5 and 3.6) it is conservative for internal criticality to assume that all precipitated solids, including mobile colloids, will be deposited inside the WP rather than transported out of the WP.
- 3.11 It is assumed that corrosion rates will not be significantly enhanced by microbiologically influenced corrosion (MIC). The bases for this assumption are that: (1) MIC will probably not occur until the repository has cooled to temperatures below 100°C (212°F) and relative humidity is above 60%, (2) although MIC may increase corrosion pit and crevice density, its effect on corrosion rate will be low, and (3) Alloy 22 has not been associated with documented cases of MIC (Ref. 7, p 3-84).
- 3.12 It is assumed that sufficient decay heat is retained within the WP over times of interest to cause convective circulation and mixing of the water inside the WP. The analysis that serves as the rationale for this assumption is discussed in Reference 11 (Attachment VI).
- 3.13 It is assumed that the reported alkalinity in analyses of J-13 well water corresponds to bicarbonate (HCO_3^-) alkalinity. Contributors to alkalinity in J-13 well water, in addition to bicarbonate, potentially include borate, phosphate, and silicate. However, at pH less than 9, the contribution of silicate will be small, and in any case the concentrations of all three of these components in J-13 well water are small. Fluoride or nitrate do not contribute to alkalinity unless a sufficiently low pH is reached. The rationale for this assumption is the observation that the calculated electrical neutrality, using the assumption, is zero, within the analytical uncertainty, as it should be. The same assumption is implicitly made in Ref. 54 (DTN: MO0006J13WTRCM.000).
- 3.14 It is assumed that the rate of entry of water into, as well as the rate of egress from, a WP is equal to the rate at which water drips onto the WP. The rationale for this assumption is that for most of the time frame of interest, i.e., long after the corrosion barriers become largely degraded, it is more reasonable to assume that all or most of the water will enter the degraded WP than to assume that a significant portion will instead be diverted around the remains. Diversion of the water with a consequent lower entry rate has not been represented by the present calculations.
- 3.15 It is assumed that the most insoluble solids for a fissile radionuclide will form. This approach is conservative with respect to internal criticality since it will lead to the

maximum retention of fissile material within the WP during EQ6 runs.

- 3.16 Even though up to two scrap baskets containing broken or damaged N Reactor SNF rods may be loaded inside an MCO (Ref. 1, Sec 4.41), it is assumed that each MCO contained either six baskets of Mark IA or five baskets of Mark IV intact N Reactor SNF rods (Ref.1, Table 4-1). The rationale for this assumption is that these two cases represent the highest U loading of the two MCOs in a WP and is therefore, conservative.
- 3.17 A number of minor assumptions have been made about the geometry of the N Reactor codisposal WP. The bases for these assumptions are outlined and referenced in the spreadsheet "N-Reactor.xls" and files "dataMIA.in" and "dataMIV.in" (Attachment III) and are also discussed in Section 5.1. These assumptions were used to represent the WP geometry with the greatest accuracy possible. Where inadequate information about WP geometry was available, or it was necessary to choose among competing representations of WP geometry, the choice that appeared to lead to the greatest precision was always chosen.
- 3.18 For any WP components that were described as "316" stainless steel, without indication of the carbon grade, the alloy was assumed to be the low-carbon equivalent (see Section 5.1.1 for nomenclature). The rationale for this assumption is that, in general, the carbon in the steel is totally insignificant compared to the carbon supplied by the fixed CO₂ fugacity of the EQ3/6 calculation, and to the constant influx of carbonate via J-13 well water.
- 3.19 The Zircaloy-2 cladding on the N Reactor fuel rods is assumed to be inert. The rationale for this assumption is that this material has low chemical reactivity at low temperatures.
- 3.20 For most of the calculations, fresh (unirradiated) fuel is used. This approach is used since unirradiated fuel has a higher fissile content than spent fuel which also has significant neutron absorption cross sections. Therefore, using the fresh fuel is more conservative for criticality.
- 3.21 Zircaloy and Zr corrosion kinetics studies (Ref. 14) revealed these materials to be resistant against chemical and biological corrosion. Recent studies on corrosion of Zircaloy-clad SNF indicate growth of oxide films for a time span of a million years to be about 7.6E-03 millimeter (0.3 mil). Given the extremely slow corrosion rate, breach of the relatively thick 20-40 mil N Reactor SNF cladding (Ref. 1, Table 3-2) during the time period of interest would probably occur only as a result of mechanical damage or defect. However, the underwater storage of N Reactor SNF has resulted in the degradation or damage of over half of the fuel assemblies (Ref. 1, Appendix C, Sec. 3). Therefore, to account for cladding defects and mechanical damage, two alternative assumptions were made regarding the fraction of the SNF that is exposed to water in the WP. In one case, it was assumed that 10% of the SNF surface area is exposed to water immediately after the WP is breached and that the cladding protects the remaining SNF from exposure to water

for the duration of the run. In the rest of the cases, it was assumed that all of the SNF is immediately exposed to water upon breach of the WP. The rationale for these assumptions is that it conservatively accounts for defects and mechanical damage during storage, shipping, or packing, and that it provides a rationale for examining the sensitivity of the results to assumptions about cladding integrity.

- 3.22 The most recent revision of Reference 36 gives the degradation rate of N Reactor SNF as $2.0833\text{E-}09 \text{ mol/cm}^2\cdot\text{s}$. For the purpose of this study, it is assumed that the original value of $2.02546 \text{ mol/cm}^2\cdot\text{s}$, upon which all the calculations are based, is correct. The basis for this assumption lies in the fact that the difference is miniscule over the time frame required for waste degradation and should, therefore, not significantly change any of the information obtained in the calculations.
- 3.23 When "time" or "years" are discussed in the calculation, this refers to the time or years after the waste package has been breached, allowing water to enter.
- 3.24 For Reference 53, the data in the DTN is adequate for what will be seen in the repository and does not need further information or qualification. The basis for this is that this is the most current data, is design related information, and subject to change.

4. USE OF COMPUTER SOFTWARE AND MODELS

This section describes the computer software used to carry out the analysis.

EQ3/6 Software Package—The EQ3/6 software package originated in the mid-1970s at Northwestern University (Ref. 22). Since 1978, Lawrence Livermore National Laboratory (LLNL) has been responsible for maintenance of EQ3/6. The software has most recently been maintained under the sponsorship of the Civilian Radioactive Waste Management Program of the United States Department of Energy (DOE). The major components of the EQ3/6 package include: EQ3NR, a speciation-solubility code; EQ6, a reaction path code which models water/rock interaction or fluid mixing in either a pure reaction progress mode or a time mode; EQPT, a data file preprocessor; EQLIB, a supporting software library; and several (>5) supporting thermodynamic data files. The software deals with the concepts of thermodynamic equilibrium, thermodynamic disequilibrium, and reaction kinetics. The supporting data files contain both standard state and activity coefficient-related data. Most of the data files support the use of the Davies or B-dot equations for the activity coefficients; two others support the use of Pitzer's equations. The temperature range of the thermodynamic data in the data files varies from 25°C only, for some species, to a full range of 0-300°C for others. EQPT takes a formatted data file (a "data0" file) and writes an unformatted near-equivalent called a "data1" file, which is actually the form read by EQ3NR and EQ6. EQ3NR is useful for analyzing groundwater chemistry data, calculating solubility limits, and determining whether certain reactions are in states of partial equilibrium or disequilibrium. EQ3NR is also required to initialize an EQ6 calculation.

EQ6 represents the consequences of exposing an aqueous solution to a set of reactants, which react irreversibly. It can also represent fluid mixing and the consequences of changes in temperature. This code operates both in a pure reaction progress frame and in a time frame. In a time frame calculation, the user specifies rate laws for the progress of the irreversible reactions. Otherwise, only relative rates are specified. EQ3NR and EQ6 use a hybrid Newton-Raphson technique to make thermodynamic calculations. This is supported by a set of algorithms that create and optimize starting values. EQ6 uses an ordinary differential equation integration algorithm to solve rate equations in time mode. The codes in the EQ3/6 package are written in FORTRAN 77 and have been developed to run under the UNIX operating system on computers ranging from workstations to supercomputers. Further information on the codes of the EQ3/6 package is provided in References 22, 23, 24, and 25.

Solid-Centered Flow-Through Mode—EQ6 Version 7.2b, as distributed by LLNL, does not contain an SCFT mode. To add this mode, it is necessary to change the EQ6 source code, and recompile the source. However, by using a variant of the "special reactant" type built into EQ6, it is possible to add the functionality of SCFT mode in a very simple and straightforward manner. This mode was added to EQ6 per Software Change Request (SCR) LSCR198 (Ref. 17), and the Software Qualification Report (SQR) for Media Number 30084-M04-001 (Ref. 18).

The new mode is induced using a new type of reactant. The EQ6 input file nomenclature for this new mode is jcode=5; in the Daveler format, it is indicated by the reactant type DISPLACER. The jcode=5 is immediately trapped and converted to jcode=2, and a flag is set to indicate the existence of the DISPLACER reactant. Apart from the input trapping, the distinction between the DISPLACER and SPECIAL reactants is seen only in one 9-line block of the EQ6 FORTRAN source code (in the reacts subroutine), where the total moles of elements in the rock plus water system (mte array) is adjusted by adding in the DISPLACER reactant, and subtracting out a commensurate amount of the total aqueous elements (mteaq array).

This new EQ6 mode acts as a substitute for the allpost/nxtinput method described in References 19 and 27.

EQ3/6 7.2bLV—EQ3/6 7.2bLV (References 12, 13, and 35) is the only version of EQ6 capable of incorporating radioactive decay, and is the only version capable of passing time-varying aqueous compositions from run to run. Although EQ6 7.2bLV was the version of EQ6 used for this calculation, these two new features were not employed.

4.1 SOFTWARE

The software package, EQ3/6, Version 7.2b (Ref 64, LLNL: UCRL-MA-110662), was approved for QA work by LLNL. An installation and testing report (Ref. 20) was written and submitted to Software Configuration Management (SCM), and the proper installation was verified, before the runs described in this calculation were made. The implementation of the SCFT mode is covered by the SCR LSCR198, and the SQR for Media Number 30084-M04-001. The SCFT addendum was installed on three of the central processing units (CPUs) identified in block 16 of the SCR, and the installation and test reports were filed and returned to SCM before the calculations were run. The version 7.2bLV (Ref. 35) of EQ6 was developed under software activity plan 10075-SAP-7.2bLV-00, the validation and test report (VTR, Ref. 12) has been reviewed and approved, and a compact disc (CD) with the software was submitted to CM. All the EQ6 runs were performed on a Sandia National Laboratory system, CPU # R433480, a Dell Optiplex G1 450 MHz Pentium II. In this study, EQ3/6 was used to provide the following:

- 1) A general overview, of the expected, chemical reactions
- 2) The degradation products from corrosion of the waste forms and canisters
- 3) An indication of the minerals, and their amounts, likely to precipitate within the WP.

The programs have been used within the range of parameters for which they have been verified and are therefore appropriate for the application. The software was used within its range of validation. However, some runs simulated periods of high ionic strength (1 to ~10). While EQ6 is capable of handling high ionic strengths, there is no Yucca Mountain Project (YMP)-qualified thermodynamic database with corrections for high ionic strength. To address this issue, several sensitivity tests were performed using other thermodynamic databases that have corrections for high ionic strength (Ref. 58, Sec. 5.1.2). The results of calculations relating to these tests have shown that calculations at high ionic strength, using the "data0.ymp" database, overestimate the

solubility of Pu and U, which is conservative with respect to external accumulations of these elements (see assumption 3.6).

The calculation inputs include several EQ6 database files with the file extension "ymp" and other EQ6 input files specific to different WP degradation scenarios with the extension "6i". There are several types of EQ6 output files and they are not all important for the purpose of this calculation. The most relevant EQ6 input and output files for this calculation are described further in Section 5 and can be found on the compact disc (CD) in Attachment III.

The EQ3/6 package has been verified by its present custodian, LLNL. The source codes were obtained from SCM in accordance with the Management & Operating Contractor (M&O) AP-SI.1Q procedure (Ref. 48). The code was installed on the Pentium PC according to an M&O-approved Installation and Test procedure (Ref. 20).

4.2 SOFTWARE ROUTINES

Spreadsheet analyses (Attachment III) were performed with Microsoft Excel Version 97 SR-2, installed on a PC. The specific spreadsheets used for results reported in this document, are included with the attached CD (Attachment III). All formulas in the spreadsheets are identified and labeled and they are used only one time so there is only one version. The formulas used in the spreadsheets have been checked against the formulas used for the calculations in the spreadsheet and have been found to be identical.

The volume and area of some of the WP components were calculated using the software routine volarea.exe, Version 1 provided in the attached CD (Attachment III). A listing of the code and instructions for running the program is provided in the file volarea.c also provided in Attachment III. Documentation of the volarea routine is provided in Attachment I. A change history is not provided because this is the first version of the routine. The program was written in ANSI C and compiled under Microsoft C++, Version 6.0. Besides calculating volumes and areas, the program computes moles of all materials described in the input file. (Volarea recognizes only a file named "data.in." as the input file for any particular run. The produced output file "data.out" and the input file must then be renamed, after the program runs, to a more fitting description of the data in question). The input files for N Reactor SNF WPs are included in Attachment III ("dataMIA.in" and "dataMIV.in"-either of these can be run again by changing the "dataMIA" or "dataMIV" to "data"). Besides the input file, another file is necessary to run the routine: "atwts.in." The output file provides the volume, area, and mole calculation results. The results of the volume, area and mole calculations for N Reactor Mark IA and Mark IV SNF WP components can be found in the spreadsheet "N-Reactor.xls", sheets "Mols_rct", "Void_calc", and "EDA2" (Attachment III). Also included in the attached CD are the output files "dataMIA.out" and "dataMIV.out" (from "data.out") that contain the results from volarea.

Some of the calculation results were extracted into text files using the program PP (part of the EQ6 reaction path code - Ref. 50), PP is a plotting routine, but it is also possible to extract the data from a plot in PP into a text file that can be imported into another program,

such as Microsoft Excel. PP is exempt from the requirements of procedure AP-SI.1Q (Ref. 48). Section 2.1.5 of the procedure states: "Software used solely for visual display or graphical representation of data which is used in a product which is checked and approved in accordance with applicable procedures and meets stated acceptance criteria is exempt."

4.3 MODELS

None used.

5. CALCULATION

The existing database supplied with the EQ3/6 computer package is sufficiently accurate for the purposes of this calculation. The data have been carefully scrutinized by many experts over the course of several decades and carefully selected by LLNL for incorporation into the database (Refs. 22, 23, 24, and 25). These databases are periodically updated and/or new databases added, such as the data0.ymp.R0 database, DTN: MO0009THRMODYN.001, that has recently been qualified for use on the YMP. This file was developed and qualified after the calculations reported here were performed. Future revisions of this calculation will employ the most recent qualified database (see Assumption 3.6). Every run of either EQ3 or EQ6 documents automatically which database is used. The databases include references for the sources of the data. The reader is referred to this documentation, included in the electronic files labeled data0 that accompany this calculation, for details (Attachment III). Most recent calculations of WP degradation have used a composite thermodynamic database called "data0.nuc.R8", derived from the "data0.skb" and other databases provided by LLNL. A preliminary version of "data0.ymp" (Attachment III) dated June 29, 2000, was used for this calculation. The version of data0.ymp that has been qualified (DTN: MO0009THRMODYN.001) for use on the YMP is only slightly altered (molar volumes and glass as a mineral were added to the database) from the version used for the calculations reported here. Therefore no significant impact of the qualification of the new data0.ymp file is anticipated. For more information on the revision history of the database, go to "data0.ymp" in Attachment III.

The calculations begin with selection of data for compositions, amounts, surface areas, and reaction rates of the various components of N Reactor SNF WPs. These quantities are recalculated to the form required for entry into EQ6. For example, weight percentages of elements or component oxides are converted to mole fractions of elements; degradation rates in micrometers/year are converted into moles per square centimeter per second, etc. Spreadsheets (Attachment III) provide details of these calculations, and the general procedure is also described in detail in Reference 27 (Section 4). The final part of the input to EQ6 consists of the composition of J-13 well water together with a rate of influx to the WP that corresponds to suitably chosen percolation rates into a drift and drip rate into a WP (Section 5.1.1.3). The EQ6 output provides the results representing the chemical degradation of the WP, or components thereof. Sometimes the degradation of the WP is divided into stages, e.g., degradation of HLW glass before breach and exposure of the SNF assemblies and basket materials to the water. The results include the compositions and amounts of solid products and of substances in solution. Details of the results are presented below.

5.1 CALCULATION INPUTS

5.1.1 WP Materials and Performance Parameters

This section provides a brief overview of the physical and chemical characteristics of N Reactor SNF WPs, and describes how the WP is represented in the EQ6 inputs. The conversion of the WP physical description, into parameters suitable for the EQ6 input files, is performed by the

spreadsheet "N-Reactor.xls". Additional details of the description may be found in Reference 1 and the references cited therein.

Material nomenclature for the stainless steels and carbon steels used throughout this document includes: SA-240 S31603 stainless steel (hereafter referred to as 316L stainless steel); Unified Numbering System (UNS) N06625 and SA-240 S30403 stainless steel (hereafter referred to as 304L stainless steel); SA-516 and SA-36 carbon steel (hereafter referred to as A516 carbon steel).

5.1.1.1 Physical and Chemical Form of the N Reactor SNF WP

It is convenient to consider the N Reactor SNF WP as several structural components, specifically:

- 1) The outer shell, consisting of Alloy 22;
- 2) The inner shell composed of 316L stainless steel;
- 3) Two, perpendicular, carbon steel (A516) A-plates which intersect to form 4 vertical compartments separating the two GPCs and the two MCOs inside the WP;
- 4) Two MCO stands composed of A516 carbon steel;
- 5) Two GPCs, the 304L containers of the solidified HLW glass;
- 6) Two MCOs composed of 304L stainless steel;
- 7) Baskets, constructed of 304 L stainless steel, five containing 54 Mark IA fuel elements each, or six containing 48 Mark IA fuel elements each; and Aluminum (Al 1100) spacers which hold the fuel elements in place, all of which are inside each MCO.
- 8) The N Reactor (U-metal) Mark IA or Mark IV SNF rods.

The details of each of the above numbered components are in the spreadsheet "N Reactor.xls" in sheets "Glass_canisters", "EDA2", "Void_calc" and "Mols_rct" (Attachment III) and the volarea input and output files ("dataMIA.in", "dataMIV.in", "dataMIA.out" and "dataMIV.out") (Attachment III); as well as in Reference 1 (Sections 3 and 4).

Table 1 provides a summary of the compositions of the principal steel alloys used in the calculations. In Table 1 and all tables from this document, the number of digits reported does not necessarily reflect the accuracy or precision of the calculation. In most tables, three to four digits after the decimal place have been retained, to prevent round-off errors in subsequent calculations.

Table 2 summarizes the reported characteristics of the N Reactor SNF. For most of the calculations, the composition of fresh (unirradiated) fuel was used. Use of the fresh fuel is conservative, since most fission products have significant neutron absorption cross sections, and the unirradiated fuel has a higher fissile content than partially spent fuel. For Case 9 (Table 8), the composition of Mark IA fuel 10 years since discharge with 16% of the Pu as ^{240}Pu , estimated using data from Table 3-8 in Reference 1 (Attachment III, spreadsheet "N-Reactor.xls", sheet "Burnup Fuel"), was used.

Table 1. Steel and Aluminum Alloy Compositions

Element	Aluminum 1100 ^a		A516 Carbon Steel ^b		304L Stainless Steel ^c		316L Stainless Steel ^d	
	Weight %	Atom Fraction	Weight %	Atom Fraction	Weight %	Atom Fraction	Weight %	Atom Fraction
Al	99.00	0.9930						
C			0.30	0.0138	0.03	0.0014	0.03	0.0014
Mn			1.03	0.0103	2.00	0.0199	2.00	0.0202
P			0.04	0.0006	0.05	0.0008	0.05	0.0008
S			0.04	0.0006	0.03	0.0005	0.03	0.0005
Si	0.45	0.0043	0.28	0.0054	0.75	0.0146	0.75	0.0148
Cr					19.00	0.1997	17.00	0.1810
Cu	0.05	0.0002						
Ni					10.00	0.0931	12.00	0.1132
Mo							2.50	0.0144
N					0.10	0.0039	0.10	0.0040
Fe	0.50	0.0024	98.33	0.9694	68.05	0.6660	65.55	0.6498
Total	100.00	1.0000	100.00	1.0000	100.00	1.0000	100.00	1.0000

Sources: ^aBased on Al 1100 composition in Reference 61 (p. 7, Table 1)

^bReference 40 (p. 321, Table 1)

^cReference 47 (p. 2, Table 1)

^dReference 52 (p. 2, Table 1)

Table 2. N Reactor (U-metal) SNF Composition

Fuel Type	Mark IA Inner Rod (Fresh Fuel) ^a	Mark IA Outer Rod (Fresh Fuel) ^a	Mark IA Inner & Outer Rod (Decayed) ^b	Mark IV Inner & Outer Rod (Fresh Fuel) ^a
Isotope	Weight %	Weight %	Weight %	Weight %
U-234			0.0070	
U-235	0.9470	1.2500	0.7423	0.9470
U-236	0.0392	0.0392	0.1185	0.0392
U-238	99.0138	98.7108	98.8451	99.0138
Np-237			0.0204	
Pu-239			0.2210	
Pu-240			0.0440	
Pu-242			0.0016	
Sum	100	100	100	100

Sources: ^aReference 1, Table 3-1

^bReference 1, Table 3-8; as calculated in Attachment III ("N-Reactor.xls"; sheet "Burnup Fuel"). This composition was used only for Case 9 (Table 8).

Table 3 gives the composition of the HLW glass used in the calculations (Ref. 31, Attachment I, p. I-7). The composition in Reference 31 was simplified to produce the values listed in Table 3 (worksheet "Composition" in spreadsheet "N-Reactor.xls" in Attachment III). Minor elements or elements with questionable thermodynamic data were removed (Ag, Cr, Cs, Cu, Li, Mn, Ni, Pb, Th, Ti, Zn), and shorter half-life Pu isotopes were "predecayed" to longer half-life U isotopes: ^{242}Pu was converted to ^{238}U ; ^{241}Pu was converted to ^{237}Np , which was converted to ^{233}U ; ^{240}Pu was converted to ^{236}U ; ^{239}Pu was converted to ^{235}U ; and ^{238}Pu was converted to ^{234}U . Since small amounts of neutron absorbers (Ag, Th, Zn) were removed in the simplified glass composition, this approach is conservative for internal criticality analyses. This simplification of the HLW glass composition allows the material to be entered as a pseudo-mineral, GlassSRL, in the EQ6 database, "data0.ymf" (Attachment III). If the HLW glass is entered in the database as a mineral, a pH dependent glass degradation rate using the EQ6 transition state theory (TST) formalism (Ref. 25, Section 3.3.3) can be applied.

Table 3. HLW Glass Composition^a and Degradation Rate Constants

Element	Weight %	Moles/100g HLW Glass ^b
O	4.2605E+01	2.7039E+00
U	1.8612E+00	7.8186E-03
Ba	1.4718E-01	1.0751E-03
Al	2.3285E+00	8.6298E-02
S	1.2849E-01	4.0071E-03
Ca	6.5021E-01	1.6224E-02
P	1.5136E-02	4.8866E-04
Si	2.1808E+01	7.7649E-01
B	3.1486E+00	2.9124E-01
F	3.1565E-02	1.6615E-03
Fe	9.6172E+00	1.7221E-01
K	2.9347E+00	7.5059E-02
Mg	8.1001E-01	3.3327E-02
Na	1.3259E+01	5.7672E-01
Total	1.0000E+02	4.7465E+00
Total Rate Constant ^c = $k_1[\text{H}^+]^{0.4} + k_2[\text{H}^+]^{0.8}$ (mol/cm ² ·s) ^d		
Moderate Rate Constant (k_1)	(liter/cm ² ·s)	8.85753E-19
High Rate Constant (k_1)	(liter/cm ² ·s)	1.07560E-17
Moderate Rate Constant (k_2)	(liter/cm ² ·s)	7.97555E-13
High Rate Constant (k_2)	(liter/cm ² ·s)	4.87424E-12

Sources: ^aSimplified composition based on Reference 31 (Attachment II, p. II-7) as calculated in spreadsheet "N-Reactor.xls", sheet "Composition" (Attachment III)

^cReference 35; (Section 6.2.3.3, Equations 7 and 8); Attachment III ("N-Reactor.xls", sheet "Rates"). rates checked against Reference 28.

NOTES: ^bThis is the composition added to "data0.ymf" for the pseudo-mineral, GlassSRL.

^dOne mole = 100g HLW glass. The molecular weight of all WP components was set to 100 g to simplify inputs to EQ6.

As was shown in Sections 5.3.2 and 5.3.3 of Reference 32, EQ6 estimates of U loss, from the WP, are not greatly affected by substantial variations in the composition of the HLW glass. The actual HLW glass composition used in the GPCs may vary significantly from these values, since the sources of the HLW glass and melting processes are not currently fixed. For example, compositions proposed for Savannah River Site HLW glass vary by a factor of ~6 in U_3O_8 content, from 0.53 to 3.16 weight percent (Ref. 33, p. 3.3-15, Table 3.3.8.). The Si and alkali metal contents (Na, Li, and K) of the HLW glass have perhaps the most significant bearing on EQ6 calculations. The amount of Si in the HLW glass strongly controls the amount of clay that forms in the WP, and the Si activity controls the presence of insoluble uranium phases such as soddyite $[(UO_2)_2SiO_4 \cdot 2H_2O]$, sodium boltwoodite $(NaUO_2SiO_3OH \cdot 1.5H_2O)$, or α -uranophane $[Ca(UO_2SiO_3OH)_2 \cdot 5H_2O]$. As the HLW glass degrades in an EQ6 run, the alkali metal content of the corrosion products increases and the pH rises. The Si and alkali metal contents in Table 3 are typical for proposed DOE HLW glasses (Ref. 31).

A pH-dependent rate for HLW glass degradation was derived from Reference 35 (Sec. 6.2.3.3, Eq. 7 and 8), and normalized in spreadsheet "N-Reactor.xls", sheet "Rates" (Attachment III). The first rate mechanism (described with k_1) in Table 3 is dominant at pH values above 7, while the second rate mechanism (described with k_2) is dominant at pH values below 7. The high glass degradation rate constants in Table 3 are those predicted at 50°C, while the moderate rate constants are those derived for degradation at 25°C (Ref. 35, Sec. 6.2.3.3, Eq. 7 and 8).

Table 4 provides molar volume, density, initial moles, reaction rate constants, and surface area for all of the materials in the N Reactor codisposal WP that were considered in the calculation. These values were calculated using volarea (Attachment III) with the geometries of the waste package components. The molar volume is the molecular weight divided by the density. For each EQ6 special reactant, the molecular weight is assumed to be 100 g/mol, which makes the calculation less complicated and has no effect on the results. The initial moles and the surface area in Table 4 are divided by the total initial void volume of 4268.479 liters to obtain normalized values based on one liter of water which is used by the EQ6 code. The details of the void volume calculation are provided in spreadsheet "N-Reactor.xls" sheet "Void_calc" (Attachment III).

Total uranium in the waste package can be obtained by adding the U in the HLW glass to the U present in the fuel. To calculate total uranium for the Mark IA and Mark IV fuel, moles from Table 4 are converted to Kg by multiplying by the initial void volume (4268.479L) and the molecular weight (100g/mole for special reactants) to obtain the Kg of U in the fuel for two MCO stands. These values were then compared to those from reference 1 (section 3.1.2) and found to be only 0.46% different. Unlike the fuel which is 100% U, the glass is only 0.78% U. To obtain the U in the glass, the moles of glass (Table 4) must be multiplied by 0.0078 to obtain the moles of U. This is then multiplied by the void volume (4268.479 L), molecular weight of U (238 g/mole), and divided by 1000 to obtain Kg of U in the glass. Total U for waste packages containing Mark IA Fuel and Mark IV Fuel is 9711.6 Kg and 12,847.6 Kg respectively

Table 4. Properties of Materials in the N Reactor SNF Codisposal WP

EQ6 Reactant Name	WP Component	V^a			m_i^b	rk^c		sk^d
		Molar Volume (cm ³ /mol)	Density (g/cm ³)	Initial Kg	Initial Moles	Rate Constant (mol/cm ² ·s)		Surface Area (cm ²)
						1	2	
GlassSRL	HLW Glass	35.09	2.85 ^e	5,767	13.51	(Table 3)	(Table 3)	729.5
MarkIA_fuel_out	Mark IA Fuel (outer rods)	5.313	18.82 ^f	3,227	14.95	2.0255E-9 ^g	Not Applicable (N/A)	233.3
MarkIA_fuel_in	Mark IA Fuel (inner rods)	5.313	18.82 ^f	6,381	7.559	2.0255E-9 ^g	N/A	93.03
MarkIV_fuel	Mark IV Fuel (inner and outer rods)	5.313	18.82 ^f	12,745	29.86	2.0255E-9 ^g	N/A	384.1
MarkIA_out_burnup	Mark IA Fuel (outer rods - decayed)	5.313	18.82 ^f	6,351	14.88	2.0255E-9 ^g	N/A	232.2
MarkIA_in_burnup	Mark IA Fuel (inner rods - decayed)	5.313	18.82 ^f	3,211	7.522	2.0255E-9 ^g	N/A	92.57
MCOMain_304L_MarkIA	MCO canister and baskets	12.59	7.94 ^h	3,781	8.857	2.516E-14 ⁱ	8.656E-12 ^j	124.2
MCOMain_304L_MarkIV	MCO canister and baskets	12.59	7.94 ^h	2,700	6.326	2.516E-14 ⁱ	8.656E-12 ^j	114.1
MCOTop_304L	MCO top	12.59	7.94 ^h	2,985	6.995	2.516E-14 ⁱ	8.656E-12 ^j	8.523
GPC_304L	GPCs	12.59	7.94 ^h	1,483	3.475	2.516E-14 ⁱ	8.656E-12 ^j	82.06
APlate_A516	A plates and MCO stands	12.74	7.85 ^k	1,223	2.867	1.798E-11 ^l	N/A	78.33
Liner_316NG	EDA II Liner	12.53	7.98 ⁿ	13,189	30.90	2.529E-14 ⁱ	5.056E-13 ^j	63.06
AI_1100_MarkIA	AI spacers	36.90	2.71 ⁿ	91.2	0.2137	2.536E-13 ⁱ	N/A	35.29
AI_1100_MarkIV	AI spacers	36.90	2.71 ⁿ	80.7	0.1890	2.536E-13 ⁱ	N/A	33.22

NOTES: ^a V equals the molecular weight (The molecular weight of all WP components was set to 100g/mol to simplify inputs to EQ6) divided by the density (g/cm³).

^b m_i equals initial mass (g) divided by 100 g/mol divided by 4268.479 liters void volume in the WP.

^c rk equals the reaction rate (g/cm²·s) divided by 100 g/mol. These rate constants must be multiplied by the normalized surface area (sk) in cm² of each WP component to calculate the actual degradation rate in 100-g mol/s of that component.

^d sk equals total surface area divided by 4268.479 liters of void volume in the WP.

Sources: ^e Based on a range of HLW glass densities in Reference 62 (p. 26 Fig. 2 and pp 54-57)

^f Reference 1 (Sec. 3.1.4, p. 22)

^g Reference 36, Section 6.3.7, Table 1 estimates an expected degradation rate of 175,000 mg/m²·d which is converted to 100g-mol/cm²·s in "N-Reactor.xls", sheet "Rates" (Attachment III).

^h Reference 60 (p. 7, Table XI)

ⁱ This rate constant was converted to rk in 100g-mol/cm²·s from a corrosion rate of 0.1 μm/year (Ref. 49, pp. 11-13) in "N-Reactor.xls", sheet "Rates" (Attachment III).

^j Rates were calculated in spreadsheet "N-Reactor.xls", "Rates" sheet (Attachment III) using Eq. 3-14 derived from Fig. 3-15 (Reference 30, Sec. 3.1.5.4.1).

^k Reference 59 (p. 9)

^l Values from Reference 29 (p. 2.2-96 through P2.2-98) were used to derive a corrosion rate of 72 μm/year in spreadsheet "A516_Rate.xls", sheets "Prob" and "Prob_Chart" (Attachment III). This rate as well as rate from Ref. 38 (p603) was converted to rk in 100g-mol/cm²·s in "N-Reactor.xls", sheet "Rates" (Attachment III).

The degradation rates in Table 4 are average or high degradation rates (indicated by "1" or "2"). The true reaction rate is obtained by multiplying rk by sk to get mol/s. Inspection of the rates shows that for a comparable surface area, the A516 carbon steel is expected to degrade much more rapidly than the stainless steels (316L and 304L).

The A-plates are composed of A516 carbon steel, and serve two purposes: they center and hold in place and separate the MCOs and GPCs; and prevent them from transmitting undue stress to each other in the event of a fall (tip-over) of the entire WP. The MCO stands are also constructed of A516. In a breach scenario, these A516 WP components will be exposed to water and corrosion before the rest of the WP, and are expected to degrade within a few hundred to a few thousand years. Precipitation of hematite (Fe_2O_3) or goethite (FeOOH) will result from the oxidation of iron in the A516 (Ref. 2).

5.1.1.2 Chemical Composition of J-13 Well Water

It was assumed that the water composition entering the WP would be the same as that for water from well J-13 (Assumptions 3.2 and 3.3). This water has been analyzed repeatedly over a span of at least 20 years (Ref. 54, DTN: MO0006J13WTRCM.000; Ref. 6, Table 4.2). The composition of J-13 well water as used in this calculation has been adjusted slightly (see assumptions 3.9 and 3.13). Tables 5 and 6 contain the EQ3NR input file constraints for J-13 well water composition and the EQ6 input file elemental molal composition for J-13 well water used for this calculation.

The "Basis Species" column of Table 5 lists the chemical species names recognized by EQ3NR and EQ6. Since some of the components of J-13 well water, as analyzed (Ref. 54, DTN: MO0006J13WTRCM.000 and Ref. 6, table 4.2), are in a different chemical form than the species listed in this column, these components must be substituted or "switched" with the basis species for input into EQ6 and are listed in the "Basis Switch" column. Basis species listed as "Trace" in the "Basis Switch" column are not found in J-13 well water, as analyzed (Ref. 54, and Ref. 6), but are in the composition of other WP components and must be input at a minimum concentration for numerical stability in EQ6 calculations.

Table 5. EQ3NR Input File Constraints for J-13 Well Water Composition

Basis Species	Basis Switch	Concentration	Units
redox		-0.7	log fO ₂
Na ⁺		4.580E+01 ^c	mg/L
SiO ₂ (aq)		6.097E+01 ^c	mg/L
Ca ⁺⁺		1.300E+01 ^c	mg/L
K ⁺		5.040E+00 ^c	mg/L
Mg ⁺⁺		2.010E+00 ^c	mg/L
Li ⁺		4.800E-02 ^d	mg/L
H ⁺		8.1 ^c	pH
HCO ₃ ⁻	CO ₂ (g)	-3 ^c	log fCO ₂
O ₂ (aq)		5.600E+00	mg/L
F ⁻		2.180E+00 ^c	mg/L
Cl ⁻		7.140E+00 ^c	mg/L
NO ₃ ⁻	NH ₃ (aq)	8.780E+00 ^c	mg/L
SO ₄ ⁻⁻		1.840E+01 ^d	mg/L
B(OH) ₃ (aq)		7.660E-01 ^d	mg/L
Al ⁺⁺⁺	Diaspore	0	Mineral
Mn ⁺⁺	Pyrolusite	0	Mineral
Fe ⁺⁺	Goethite	0	Mineral
HPO ₄ ⁻⁻		1.210E-01 ^d	mg/L
Ba ⁺⁺	Trace	1.000E-16	Molality ^a
CrO ₄ ⁻⁻	Trace	1.000E-16	Molality ^a
Cu ⁺⁺	Trace	1.000E-16	Molality ^a
MoO ₄ ⁻⁻	Trace	1.000E-16	Molality ^a
Ni ⁺⁺	Trace	1.000E-16	Molality ^a
Np ⁺⁺⁺⁺ ^b	Trace	1.000E-16	Molality ^a
Pu ⁺⁺⁺⁺ ^b	Trace	1.000E-16	Molality ^a
UO ₂ ⁺⁺	Trace	1.000E-16	Molality ^a

NOTES: ^aA trace concentration (1.0E-16 molal) is added for elements that are not in J-13 well water as analyzed, but are in the composition of the WP components, to ensure numerical stability in EQ3/6 runs.

^bOnly included for Case 9 in Table 8.

Sources: ^cRef. 54 (DTN: MO0006j13WTRCM.000)

^dRef. 6, Table 4.2

Table 6. EQ6 Input File Elemental Molal Composition for J-13 Well Water

Element	Mole/kg	Element	Mole/kg
O	5.55E+01	Mg	8.27E-05 ^a
Al	2.55E-08	Mn	3.05E-16
B	1.24E-05	Mo	1.00E-16
Ba	1.00E-16	N	1.42E-04 ^a
Ca	3.24E-04 ^a	Na	1.99E-03 ^a
Cl	2.01E-04 ^a	Ni	1.00E-16
Cr	1.00E-16	S	1.92E-04 ^a
Cu	1.00E-16	Si	1.02E-03 ^a
F	1.15E-04 ^a	U	1.00E-16
Fe	3.60E-12	K	1.29E-04 ^a
H	1.11E+02	Li	6.92E-06
Np	1.00E-16	C	2.09E-03 ^a
Pu	1.00E-16	P	1.26E-06

Sources: ^aRef. 54 (DTN:MO0006J13WTRCM.000)
Ref. 6, Table 4.2

5.1.1.3 Drip Rate of J-13 Well Water into a WP

It is assumed (Assumption 3.14) that the drip rate onto a WP is the same as the rate at which water flows through the WP. The drip rate is taken from a correlation between percolation rate and drip rate (Ref. 39, Tables 2-55 and 2-56). Specifically, percolation rates of 40 mm/year and 8 mm/year correlate with drip rates onto the WP of 0.15 m³/year and 0.015 m³/year, respectively. The choice of these particular percolation and drip rates is discussed in detail in Reference 27 (Section 5.1.1.3, p. 19).

For the present study, the range of allowed drip rates included an upper value of 0.015 m³/yr and a lower value of 0.0015 m³/yr. The upper value corresponds to the mean value for the percolation rate of 8 mm/year (Ref. 39, Figure 2.3-108), and the lower value is simply 1/10th of the upper value.

5.1.1.4 Densities and Molecular Weights of Solids

For input to criticality calculations, one must convert moles of solids to volume of solids. A few solid phases contribute the overwhelming bulk of the total volume; Table 7 provides some of the densities and molar volumes for these phases. The EQ6 database used for this calculation ("data0.ymf", Attachment III) did not contain molar volumes for some of the solids that were predicted to form during the EQ6 runs for the cases in Table 8. Molar volumes for these solids were calculated from the molecular weights of the solids in "data0.ymf" and the solid densities from various sources, as noted in Table 7. The current version of EQ6 (Section 4) performs the volume calculations for each element automatically.

Table 7. Densities, Molecular Weights and Molar Volumes of Precipitated Solids

Solid	Density (kg/m ³)	Molecular Weight (g/mole) ^c	Molar Volume (cm ³ /mol) ^c	Calculated Density (g/cm ³) ^f
Gibbsite [Al(OH) ₃]		78.004	31.956	2.441
Hematite (Fe ₂ O ₃)	5260 ^a	159.688	30.274	5.275
Pyrolusite (MnO ₂)	5060 ^a	86.937	17.181	5.060 ^a
Goethite (FeOOH)		88.852	20.820	4.268
Trevorite (NiFe ₂ O ₄)	5165 ^a	234.382	45.379 ^a	5.165 ^a
Nontronite-Ca		424.293	131.100	3.236
Nontronite-K		430.583	135.270	3.183
Nontronite-Mg		421.691	129.760	3.250
Nontronite-Na		425.267	132.110	3.219
Fluorapatite [Ca ₅ (PO ₄) ₃ F]	3200 ^a	504.302	157.594 ^b	3.2 ^a
Hydroxylapatite [Ca ₅ (PO ₄) ₃ OH]	3080 ^a	502.311	163.088 ^b	3.08 ^a
Mesolite	2259 ^a	387.783	171.661 ^b	2.259 ^a
Ni ₃ (PO ₄) ₂	4396 ^b	366.023	83.263 ^b	4.396 ^b
α-Uranophane [Ca(UO ₂ SiO ₃ OH) ₂ ·5H ₂ O]	3830 ^a	856.392	223.601 ^b	3.83 ^a
(UO ₂) ₃ (PO ₄) ₂ ·6H ₂ O	3500 ^b	1108.118	316.605 ^b	3.5 ^b
Na ₄ UO ₂ (CO ₃) ₃	3630 ^b	542.013	149.305 ^b	3.63 ^b
CaUO ₄	7459 ^b	342.105	45.865 ^b	7.459 ^b
NpO ₂	11110 ^d	268.999	24.220	11.11 ^d
PuO ₂	11.581 ^e	275.999	23.830	11.581 ^e

Sources: ^aReference 42, pp. 289 (fluorapatite), 356 (hematite), 389 (hydroxylapatite), 547 (mesolite), 704 (pyrolusite), 880 (trevorite), and 903 (α-uranophane).

^bReference 56, JCPDS cards for Ni₃(PO₄)₂ (38-1473), (UO₂)₃(PO₄)₂·6H₂O (30-1405), CaUO₄ (44-583), Na₄UO₂(CO₃)₃ (11-81).

^cAttachment III (EQ3/6 Data base, "data0.ymp") unless otherwise noted.

^dReference 55, (p. B101).

^eReference 57, (p.C-103).

NOTES: ^fCalculated from the molecular weight and molar volume unless other wise noted.

^gCalculated from the molecular weight and density.

5.1.1.5 Atomic Weights

Atomic weights were taken from References 43 and 44 (Chart of the Nuclides), and are listed in spreadsheet "N-Reactor.xls" sheet "Atomic Weights_New" (Attachment III).

5.2 DATA CONVERSION

The data presented in Section 5.1 are not in a form suitable for entry into EQ3/6. The transformation to EQ3/6 format involves converting mass fractions to mole fractions; normalizing surface areas, volumes, and moles to 1 liter reactive water in the system; and

converting rates to $\text{mol}/\text{cm}^2\cdot\text{s}$. Most of these conversions are straightforward and are performed in the spreadsheets that are included in the attached CD for this document (Attachment III). Reference 27 (Section 4) describes the conversion process in detail.

5.3 EQ6 CALCULATIONS AND SCENARIOS REPRESENTED

The rationale for selection of scenarios in EQ6 simulations is to provide conservative criticality assessments of solubility and transport of fissile materials (i.e., U, Pu, or Np compounds) in the WP. An internal criticality is possible if the fissile material remains behind in the WP. Soluble U, Pu, and Np carbonate complexes will form in the high pH solutions produced when the HLW glass degrades.

The “*Disposal Criticality Analysis Methodology Topical Report*” document defines the internal and external degradation scenarios for disposal criticality analysis (Ref. 46, pp. 3-9 through 3-12). The internal degradation configurations are based on the assumption that groundwater drips onto the upper surface of the WP and penetrates it. Groundwater accumulates inside the WP, which could dissolve and flush the SNF from the WP. Following is a summary of three groups of degradation configurations from Reference 46:

1. WP internals degrade faster than the waste forms
2. WP internals degrade at the same rate as the waste form
3. WP internals degrade slower than the waste forms.

The WP internals include all components within the WP, except SNF. The waste forms refer to SNF. The above configurations set the framework in which EQ6 scenarios could be developed. The scenarios are based on a sequence of chemical reactions as a function of time and can be divided into two general categories: single stage cases and two-stage cases.

Single-Stage Cases—In these calculations, all WP internals, including SNF, come in contact with groundwater simultaneously. These cases correspond to an extreme in which the zirconium cladding is breached immediately, thereby exposing all or a portion of the spent fuel as soon as the WP corrosion barriers are breached. These cases result in the highest dissolved radionuclide levels, and might provide the most conservative estimate of fissile material loss.

Two-Stage Cases—These EQ6 calculations start with the breach of the WP allowing groundwater to come in contact with WP internals outside the MCOs (“stage A”); during this stage the MCOs remain intact. The second stage (“stage B”) starts with breach of the MCOs and interaction of groundwater with material inside the MCOs, as well as waste forms and unaltered reactants remaining from stage A. These cases were designed to produce the lowest possible pH, by first exposing the HLW glass to J-13 well water to remove alkalinity, prior to exposure of the SNF in the second stage.

In total, 10 cases of single- and two-stage EQ6 simulations with different steel and HLW glass degradation rates, as well as varied water fluxes through the WP, were run. These cases are discussed in the following sections.

5.3.1 EQ6 Run Conditions and Nomenclature

The EQ6 codes were used to run the 10 cases summarized in Table 8. In general each case could be classified as single- or two-stage. Cases 1-9 are single-stage, and involve simultaneous exposure of the SNF and the WP materials to J-13 well water. Considering that the SNF rods are within zirconium cladding, for a conservative approach, it was assumed (Assumption 3.20) that cladding is fully breached immediately after contact with water.

The "File Names" column in Table 8 gives the root file names used to describe the runs. The EQ6 input files corresponding to these runs end with the extension ".6i" (e.g., "N40h1112.6i" is an EQ6 input file name for Case 3); these input files are included in the attached CD accompanying this calculation (Attachment III). Each EQ6 run has associated tab-delimited text files, also included in the attached CD (e.g., "N40h1112.elem_aqu.txt" for Case 3). The text files list total moles of elements in the aqueous phase ("?.elem_aqu.txt"), total moles of each element precipitated as minerals ("?.elem_min.txt"), and total moles of each element ("?.elem_tot.txt"), which is the sum of moles in the aqueous and mineral phases plus the unreacted, or remaining moles of reactants (WP components). Since the HLW glass WP component was entered into the "data0.ymp" file as a pseudo-mineral for this calculation, the remaining moles of this reactant were not included in the "?.elem_tot.txt" files. Several input files, corresponding to separate EQ6 runs, may be grouped into a "Case". Most of the important run conditions could be inferred from the root file name. Evaluation of root file names for most cases from left to right is as follows:

The first letter "N" corresponds to N Reactor SNF.

The second character (first digit after "N") is "4" for cases with Mark IV N Reactor SNF or "1" for cases with Mark IA N Reactor SNF.

The third character (second digit after "N") corresponds to revision or continuation of input file for the single- and two-stage cases; for each case, the numbers range from 0 to 2. Runs that do not converge usually require removal of the exhausted reactants and restart of the run.

The fourth character (third digit after "N") in the two-stage runs are "A" for the first stage or "B" for the second stage of the run.

The fourth character in the single-stage cases corresponds to special run conditions which are defined in the note beneath Table 8.

The fifth digit is 1 or 2 corresponding to the average or high rates of steel corrosion in Table 4.

The sixth digit in this block is 1, 2, or 0, with 1 and 2 corresponding to the moderate and high glass corrosion rates listed in Table 3; and 0 corresponding to no HLW glass present in the EQ6 run.

The seventh digit in the block is 1 or 0, with 1 corresponding to SNF dissolution rate in Table 4, and 0 corresponding to no SNF present in the EQ6 run.

The last digit in the block encodes the choice of drip rate, with 1 and 2 corresponding to $0.0015 \text{ m}^3/\text{year}$ and $0.015 \text{ m}^3/\text{year}$, respectively.

5.3.2 Examination of Cases

Table 8 summarizes all the cases run, as well as the total percentage of U (plus Pu and Np for Case 9) loss at the end of the EQ6 runs. These losses were calculated in the spreadsheet "Nreactor_U.xls", sheet "U-losses" (Attachment III). The complete output tables (aqueous, mineral, and total moles) for all the cases are included in the attached CD, as text files (Attachment III). A summary of the files included in the attached CD is given in Attachment II.

The losses calculated above were then checked by hand calculations of the areas beneath the U curve on Figures 1 through 23. These calculations produced numbers close to those calculated in the spreadsheet "Nreactor_U.xls" confirming that the % loss of U from the waste packages is very low and the numbers presented are correct. Case 5 is used as an example of how this hand calculation was executed –Attachment III, "Nreactor_U.xls" sheet "check"

Tables 9 through 27 illustrate the limits of system behavior for the single-stage runs. Single stage runs show U (plus Pu and Np for Case 9) loss from the SNF and HLW glass to alteration products and solution as a function of reaction time, and how each output varies depending upon input steel corrosion rates, HLW glass corrosion rates, SNF corrosion rates, and fluid flow rates. The two-stage run (Case 10) provides information on how the system might behave under a number of extreme scenarios. Examination of the results in Table 8 reveals the following generalizations about U (plus Pu and Np for Case 9) release from the WP.

1. Predicted total losses of U from the WP were less than 2% for all of the cases shown in Table 8. Uranium loss can be attributed to the formation of four major minerals including Schoepite $[\text{UO}_3 \cdot 2\text{H}_2\text{O}]$, Sodium Boltwoodite $[\text{NaUO}_2\text{SiO}_3\text{OH} \cdot 1.5\text{H}_2\text{O}]$, $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 6\text{H}_2\text{O}$, and α -Uranophane $[\text{Ca}(\text{UO}_2\text{SiO}_3\text{OH})_2 \cdot 5\text{H}_2\text{O}]$ with Schoepite being the most dominant.
2. Case 10, the two-stage run, predicted that all of the U in the HLW glass (1.10% of the total U in the WP) would be lost from the WP for conditions of average steel, high HLW glass degradation rates and low drip rate simulated by the first stage of the run. Only 0.02% of the total U in the WP (from Mark IA SNF only) was predicted to be lost during

the conditions simulated by the second stage of Case 10, with low steel, average SNF degradation rates, and low drip rate.

3. Cases 2 and 5 had predicted total U losses from the WP of close to 1%. Both these cases simulated conditions of high steel, average fuel, and low glass degradation rates, with a low drip rate.
4. Predicted loss of U from the WP was very low (0-0.16%) for the rest of the Cases shown in Table 8.
5. Case 9 was run assuming a decayed composition for N Reactor Mark IA SNF (Table 2). Although predicted total loss of U was very low for this case (0.02%), predicted total loss of Np was high (~83%) and predicted total loss of Pu was low (<2%).

Table 8. Summary of Cases Run, Associated Input File Names, Percent Fuel Surface Exposed to Corrosion, Percent U Loss, and Fe Oxide Corrosion Product

Case	File Names ^b	% Fuel Surface Area	Mark IV SNF	Mark IA SNF	% U Loss ^a	% Np Loss ^a	% Pu Loss ^a
1	N40h1111	100	√		0.03	N.A. ^c	N.A.
2	N40h2111	100	√		0.75	N.A.	N.A.
3	N40h1112	100	√		0.12	N.A.	N.A.
4	N10h1111	100		√	0.02	N.A.	N.A.
5	N10h2111	100		√	0.98	N.A.	N.A.
6	N10h1112	100		√	0.16	N.A.	N.A.
7	N10#1111	10		√	0.02	N.A.	N.A.
8	N10&1111	0		√	0.00	N.A.	N.A.
9 ^d	N10h1211	100		√	0.02	82.81	1.58
10	N10A1201	0		√	1.10	N.A.	N.A.
	N10B1011	100			0.02	N.A.	N.A.

NOTES: ^a U, Pu and Np losses are a percentage of total moles of U, Pu and Np in SNF and HLW glass. Pu and Np are only relevant to case 9, because the other cases use only fresh fuel.

^b Explanation of special symbols in root file names:

note: hematite was not suppressed in any of the EQ6 runs.

h = normal run without any special conditions applied

= 100% fuel moles and 10% fuel surface area exposed to corrosion

& = 100% fuel moles and 0% fuel surface area exposed to corrosion

A = First stage of a two-stage run

B = Second stage of a two-stage run

^c Not Applicable (fresh fuel contains no Pu or Np)

^d Case 9 was run with the decayed fuel compositions in Table 2.

Table 9. Predicted Elemental Composition of Corrosion Products (kg), Total Mass (kg) and Density in Selected Years for Case 1 (n40h1111)

Years	2354	112760	634010
Element			
O	4.932E+03	7.221E+03	1.195E+04
Al	2.784E-01	1.216E+01	6.449E+01
B	1.993E-14	6.389E-14	0.000E+00
Ba	1.760E-02	7.705E-01	4.103E+00
Ca	1.293E-01	4.859E+00	2.753E+01
Cl	0.000E+00	0.000E+00	2.150E-13
Cr	0.000E+00	2.917E-01	1.553E+00
Cu	1.876E+02	1.806E+02	1.695E+02
F	0.000E+00	0.000E+00	5.181E-13
Fe	1.349E+03	5.904E+03	1.417E+04
H	2.159E+02	2.188E+02	2.313E+02
C	0.000E+00	0.000E+00	6.315E-12
P	5.267E-01	3.635E+00	9.535E+00
K	5.942E-02	3.433E+00	1.968E+01
Li	0.000E+00	0.000E+00	1.781E-15
Mg	9.672E-02	3.968E+00	2.137E+01
Mn	1.706E+01	1.514E+02	3.969E+02
Mo	3.095E-01	1.163E+01	6.240E+01
N	0.000E+00	0.000E+00	9.702E-14
Na	1.939E-02	8.347E-01	7.262E+00
Ni	0.000E+00	3.825E+01	4.129E+02
S	4.110E-03	0.000E+00	1.943E-11
Si	8.096E+00	1.795E+02	8.068E+02
U	1.275E+04	1.275E+04	1.279E+04
Total (kg)	1.946E+04	2.669E+04	4.115E+04
Density (g/cm³)	4.921	4.929	4.848

Table 10. Predicted Solution Elemental Composition (mole kg⁻¹) and pH in Selected Years for Case 1 (n40h1111)

Years	2354	112760	634010
pH	5.54	5.87	6.68
Element			
Al	1.716E-10	1.177E-11	2.089E-14
B	5.617E-03	8.359E-03	9.416E-03
Ba	5.050E-07	5.961E-07	3.387E-07
Ca	1.525E-04	6.426E-05	9.171E-05
Cl	2.014E-04	2.014E-04	2.014E-04
Cr	1.223E-01	5.382E-02	7.026E-03
Cu	1.164E-03	2.338E-04	3.729E-06
F	1.475E-04	1.635E-04	1.696E-04
Fe	9.707E-12	5.720E-12	1.691E-12
C	3.130E-05	4.781E-05	1.199E-04
P	1.687E-08	2.906E-08	1.282E-07
K	1.277E-03	1.776E-03	1.173E-03
Li	3.918E-06	6.915E-06	6.915E-06
Mg	8.200E-05	1.265E-04	5.256E-05
Mn	1.584E-10	3.231E-11	4.857E-13
Mo	1.577E-03	3.007E-03	8.157E-04
N	2.592E-03	1.301E-03	2.796E-04
Na	1.291E-02	1.838E-02	1.236E-02
Ni	6.124E-02	2.423E-02	3.626E-04
S	2.473E-03	4.582E-04	3.388E-04
Si	1.493E-04	1.856E-04	1.859E-04
U	1.771E-05	9.097E-06	8.808E-06

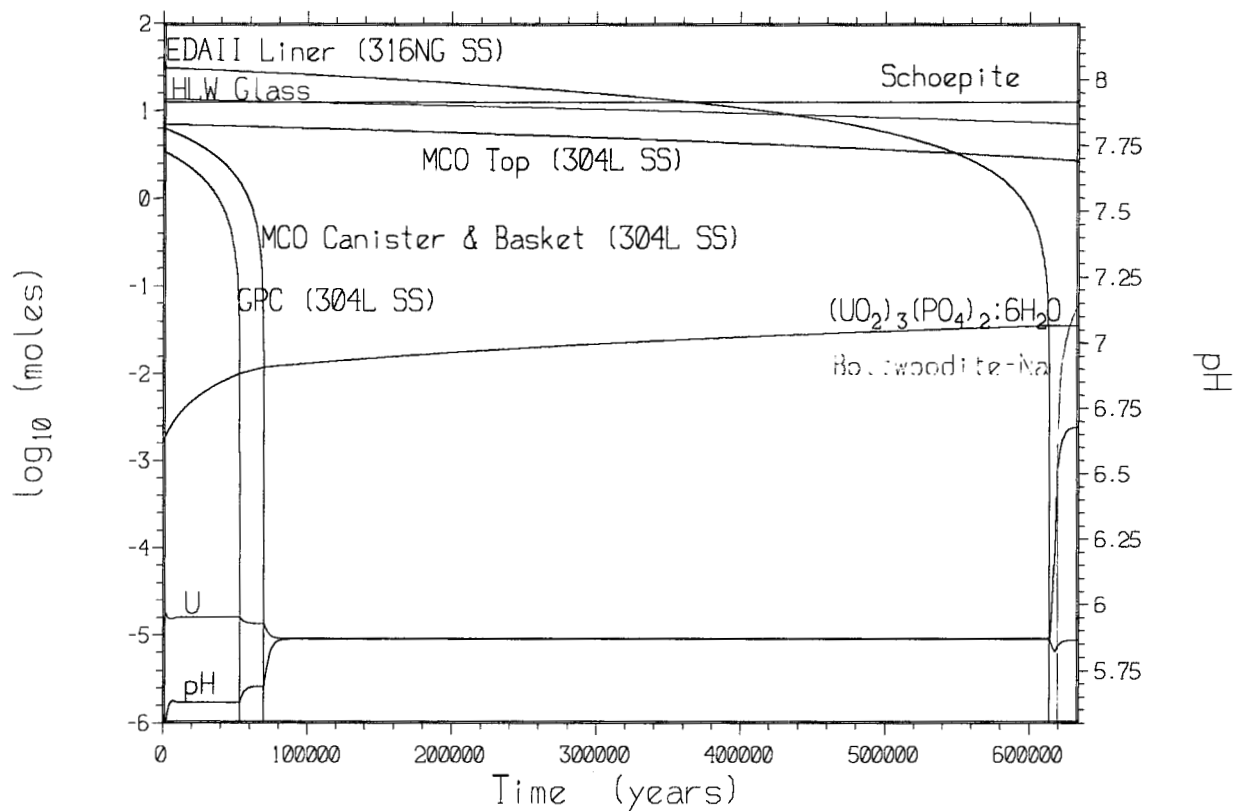


Figure 1. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time During Case 1 (n40h1111)

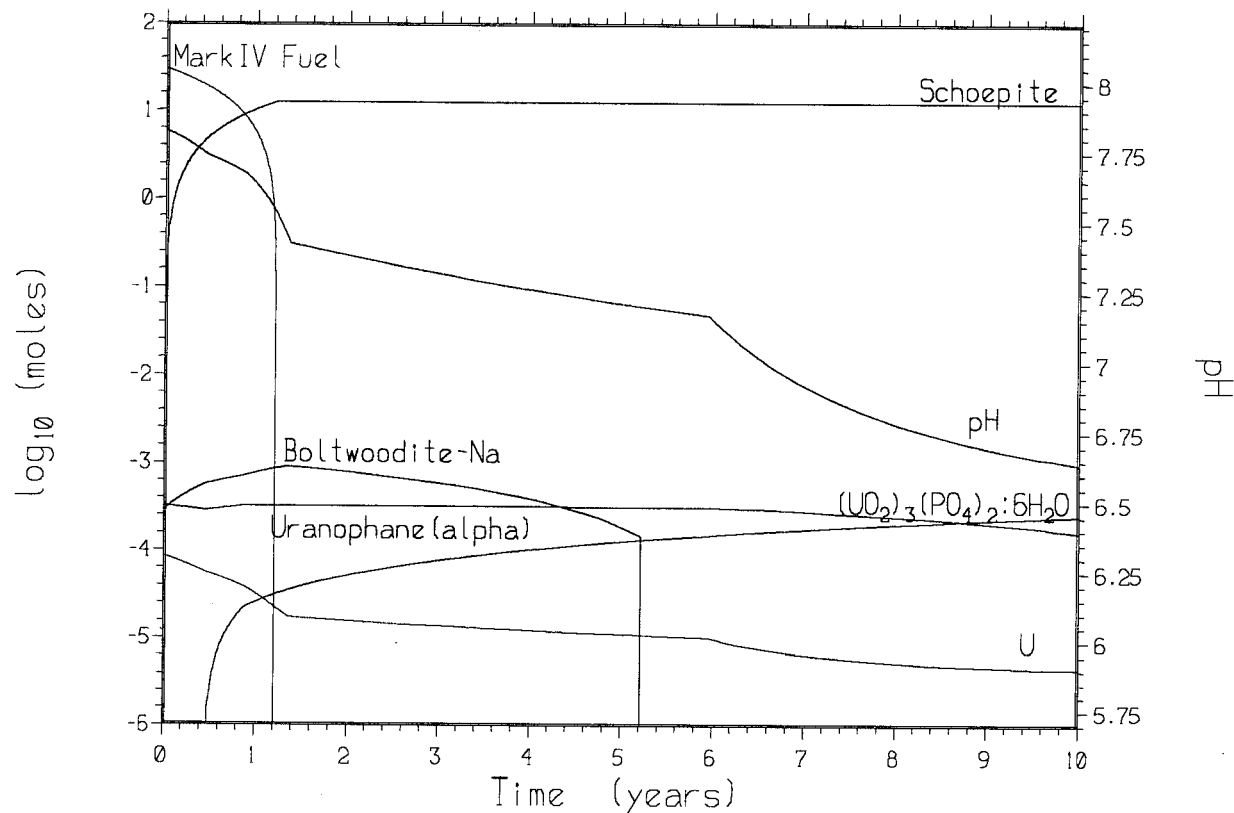


Figure 2. Predicted Concentration of N-Reactor SNF, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time Early in Case 1(n40h1111)

Table 11. Predicted Elemental Composition of Corrosion Products (kg), Total Mass (kg) and Density in Selected Years for Case 2 (n40h2111)

Years	153	106000	634260
Element			
O	5.898E+03	1.151E+04	1.345E+04
Al	9.856E-02	2.127E+01	1.343E+02
Ba	5.794E-03	1.366E+00	8.696E+00
Ca	7.518E-02	7.889E+00	4.945E+01
Cl	4.343E-11	1.231E-17	0.000E+00
Cr	1.601E-05	9.749E-25	2.189E-29
Cu	6.081E+00	1.364E+02	1.364E+02
F	8.017E-11	3.957E-17	2.086E+00
Fe	3.723E+03	1.478E+04	1.525E+04
H	2.061E+02	2.217E+02	2.393E+02
C	3.467E-12	1.195E-01	7.605E-01
P	2.097E+00	9.726E+00	1.020E+01
K	1.561E-12	1.268E+01	3.485E+01
Mg	1.575E-11	6.532E+00	4.134E+01
Mn	8.690E+01	4.199E+02	4.199E+02
Mo	1.800E-01	0.000E+00	9.086E-29
N	2.057E-08	0.000E+00	0.000E+00
Na	7.678E-12	7.951E+01	7.225E+02
Ni	7.884E-08	9.881E+01	9.879E+01
S	1.024E+00	1.380E-15	0.000E+00
Si	3.255E+01	3.922E+02	1.472E+03
U	1.274E+04	1.276E+04	1.276E+04
Total (kg)	2.269E+04	4.046E+04	4.482E+04
Density (g/cm ³)	5.000	4.964	4.680

Table 12. Predicted Solution Elemental Composition (mole kg⁻¹) and pH in Selected Years for Case 2 (n40h2111)

Years	153	106000	634260
pH	3.74	7.87	8.21
Element			
Al	3.500E-06	8.884E-16	1.038E-07
B	2.855E-03	2.401E-02	1.239E-05
Ba	1.035E-06	1.681E-08	3.687E-09
Ca	5.316E-05	9.523E-07	5.610E-06
Cl	2.014E-04	2.014E-04	2.014E-04
Cr	3.084E+00	3.593E-13	1.000E-16
Cu	1.229E-01	8.864E-08	7.478E-08
F	1.313E-04	2.548E-04	1.147E-04
Fe	3.197E-10	1.131E-12	1.138E-12
C	2.437E-06	1.343E-03	3.752E-03
P	1.218E-08	6.676E-06	1.156E-06
K	8.629E-04	2.710E-03	2.522E-03
Li	3.610E-07	6.915E-06	6.915E-06
Mg	4.077E-04	2.870E-04	2.362E-04
Mn	1.126E-07	1.702E-15	5.594E-16
Mo	3.465E-03	1.981E-09	1.000E-16
N	6.051E-02	1.416E-04	1.416E-04
Na	7.621E-03	7.467E-04	1.829E-03
Ni	1.446E+00	1.181E-06	2.601E-07
S	3.802E-03	5.211E-04	1.915E-04
Si	7.666E-05	1.893E-04	3.689E-05
U	7.802E-03	1.026E-04	5.085E-04

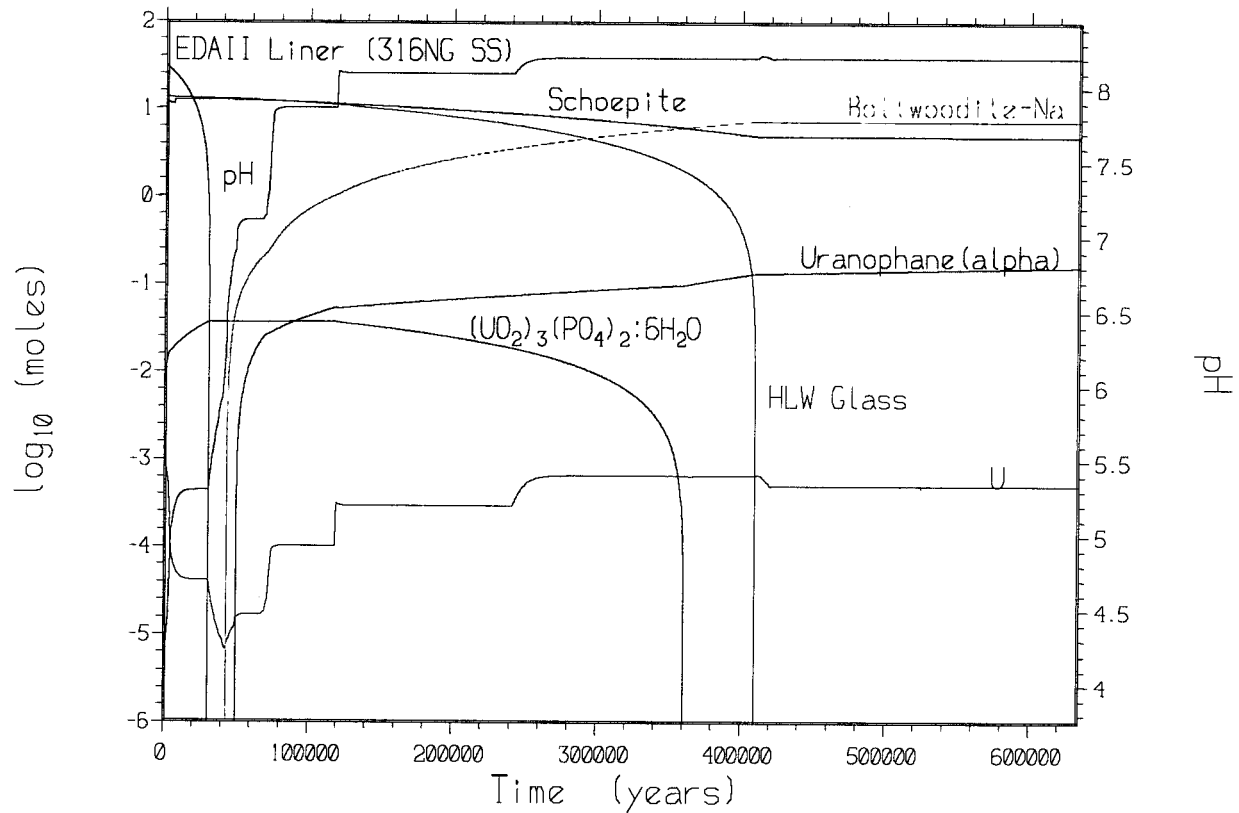


Figure 3. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time During Case 2 (n40h2111)

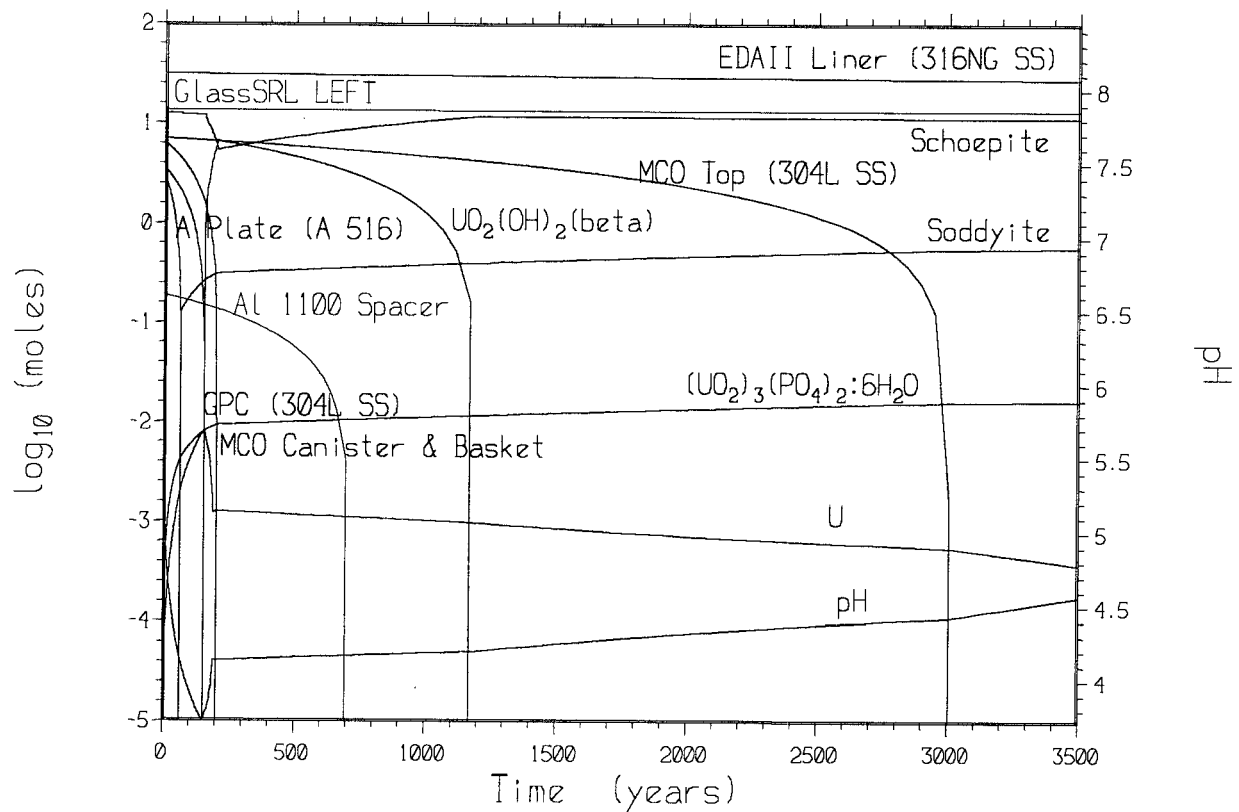


Figure 4. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time Early in Case 2 (n40h2111)

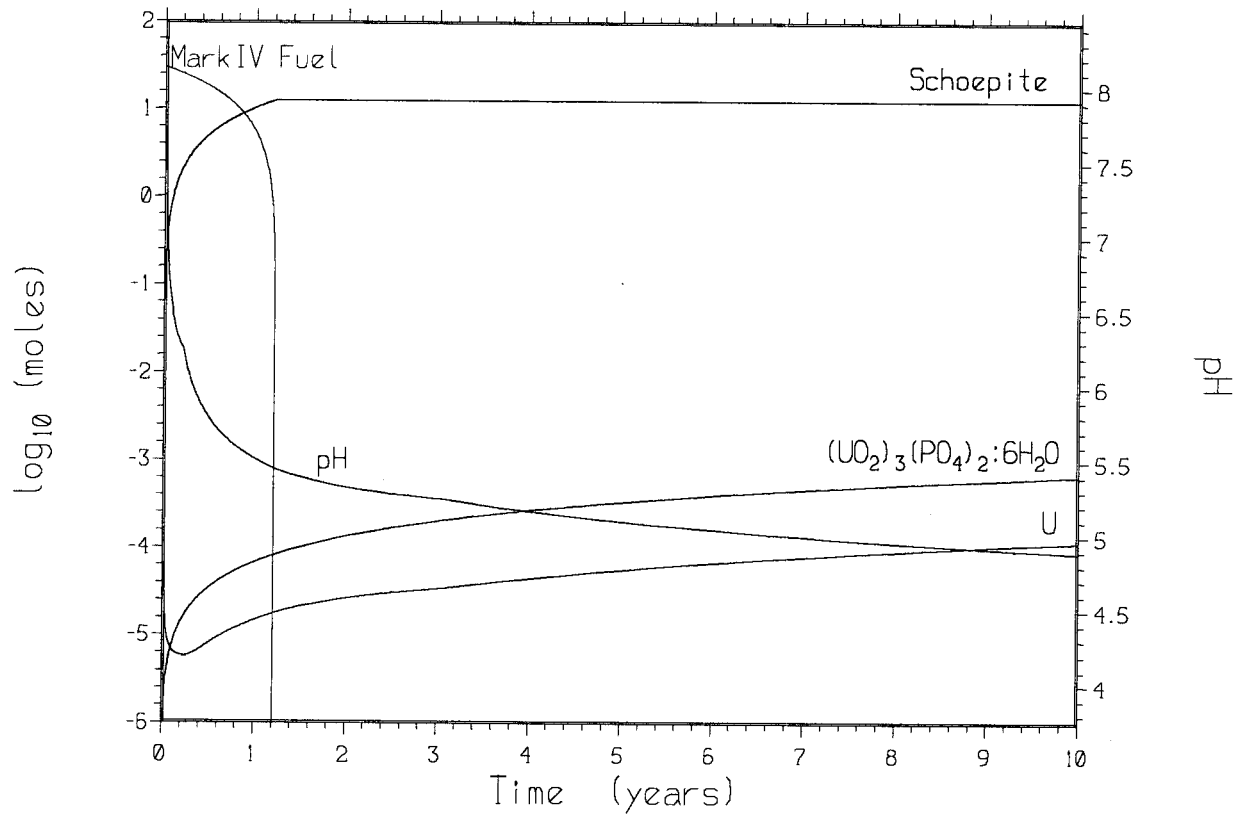


Figure 5. Predicted Concentration of N-Reactor SNF, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time Early in Case 2 (n40h2111)

Table 13. Predicted Elemental Composition of Corrosion Products (kg), Total Mass (kg) and Density in Selected Years for Case 3 (n40h1112, n41h1112, n42h1112)

Years	318	51325	101090	633790
Element				
O	4.844E+03	6.410E+03	7.127E+03	1.216E+04
Al	3.679E-02	5.201E+00	1.002E+01	6.255E+01
B	2.997E-15	1.501E-17	0.000E+00	0.000E+00
Ba	2.154E-03	2.603E-01	5.101E-01	3.222E+00
Ca	2.639E-02	3.023E+00	6.998E+00	5.738E+01
Cr	0.000E+00	9.856E-02	1.931E-01	2.406E-17
Cu	8.393E+01	1.779E+02	1.757E+02	1.651E+02
Fe	1.223E+03	4.368E+03	5.708E+03	1.417E+04
H	2.159E+02	2.171E+02	2.182E+02	2.325E+02
C	1.189E-10	1.440E-10	1.811E-10	2.818E-01
P	4.419E-01	2.591E+00	3.544E+00	9.840E+00
Mg	5.704E-04	5.098E-01	1.239E+00	1.171E+01
Mn	1.337E+01	1.061E+02	1.457E+02	3.968E+02
Mo	0.000E+00	0.000E+00	3.823E+00	2.629E+01
Ni	0.000E+00	0.000E+00	2.906E+01	5.113E+02
S	6.381E-02	1.484E-17	0.000E+00	1.484E-17
Si	4.356E+00	1.080E+02	1.881E+02	9.878E+02
U	1.275E+04	1.275E+04	1.275E+04	1.278E+04
Total (kg)	1.913E+04	2.415E+04	2.637E+04	4.157E+04
Density (g/cm ³)	4.912	4.935	4.925	4.807

Table 14. Predicted Solution Elemental Composition (mole kg⁻¹) and pH in Selected Years for Case 3 (n40h1112, n41h1112, n42h1112)

Years	318	51325	101090	633790
pH	5.58	5.84	6.27	7.06
Element				
Al	8.193E-11	7.232E-12	1.830E-13	4.331E-15
B	6.857E-04	8.557E-04	8.200E-04	1.231E-03
Ba	2.315E-07	7.311E-07	6.285E-07	7.046E-07
Ca	2.718E-04	2.736E-04	2.193E-04	3.762E-05
Cl	2.014E-04	2.014E-04	2.014E-04	2.014E-04
Cr	1.513E-02	2.158E-02	5.383E-03	7.037E-04
Cu	6.115E-04	2.032E-04	2.167E-05	6.131E-07
F	1.187E-04	1.197E-04	1.195E-04	1.219E-04
Fe	8.077E-12	5.618E-12	2.617E-12	1.317E-12
C	3.512E-05	4.648E-05	6.662E-05	2.324E-04
P	1.758E-08	2.685E-08	5.037E-08	3.448E-07
K	3.028E-04	3.467E-04	3.375E-04	4.435E-04
Li	4.849E-06	6.915E-06	6.915E-06	6.915E-06
Mg	1.543E-04	1.518E-04	1.266E-04	2.200E-05
Mn	8.062E-11	2.805E-11	2.940E-12	6.802E-14
Mo	2.616E-04	3.731E-04	2.944E-04	1.347E-03
N	4.448E-04	5.741E-04	2.576E-04	1.554E-04
Na	3.331E-03	3.662E-03	3.592E-03	4.405E-03
Ni	7.581E-03	1.081E-02	2.199E-03	5.077E-05
S	1.575E-03	2.598E-04	2.178E-04	2.101E-04
Si	1.606E-04	1.857E-04	1.857E-04	1.863E-04
U	1.441E-05	8.418E-06	6.349E-06	1.529E-05

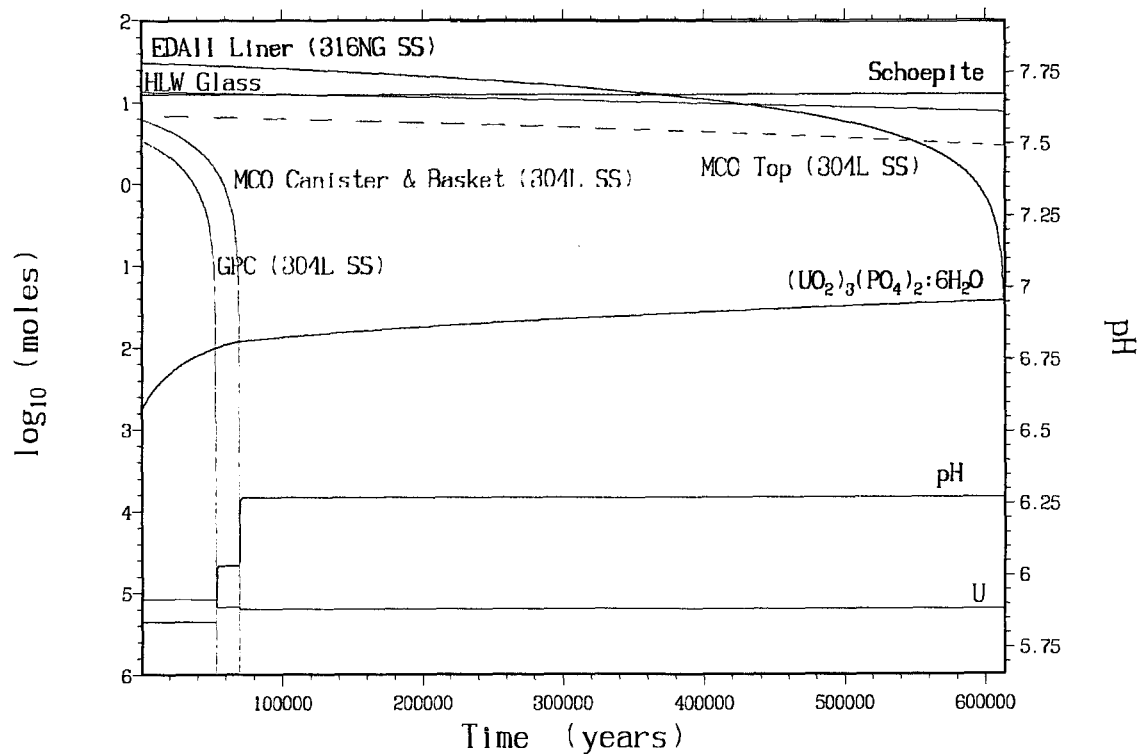


Figure 6. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time During Case 3 (n40h1112)

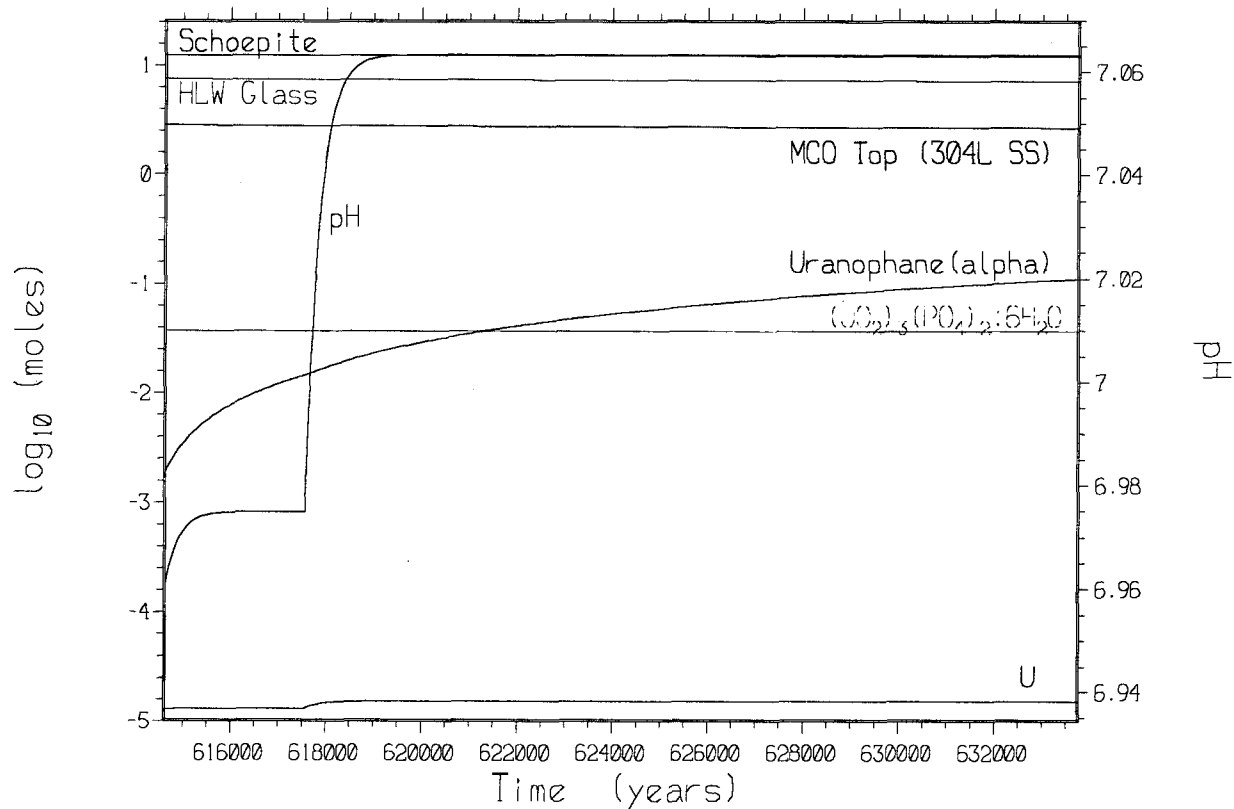


Figure 7. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time at the End of Case 3 (n42H1112)

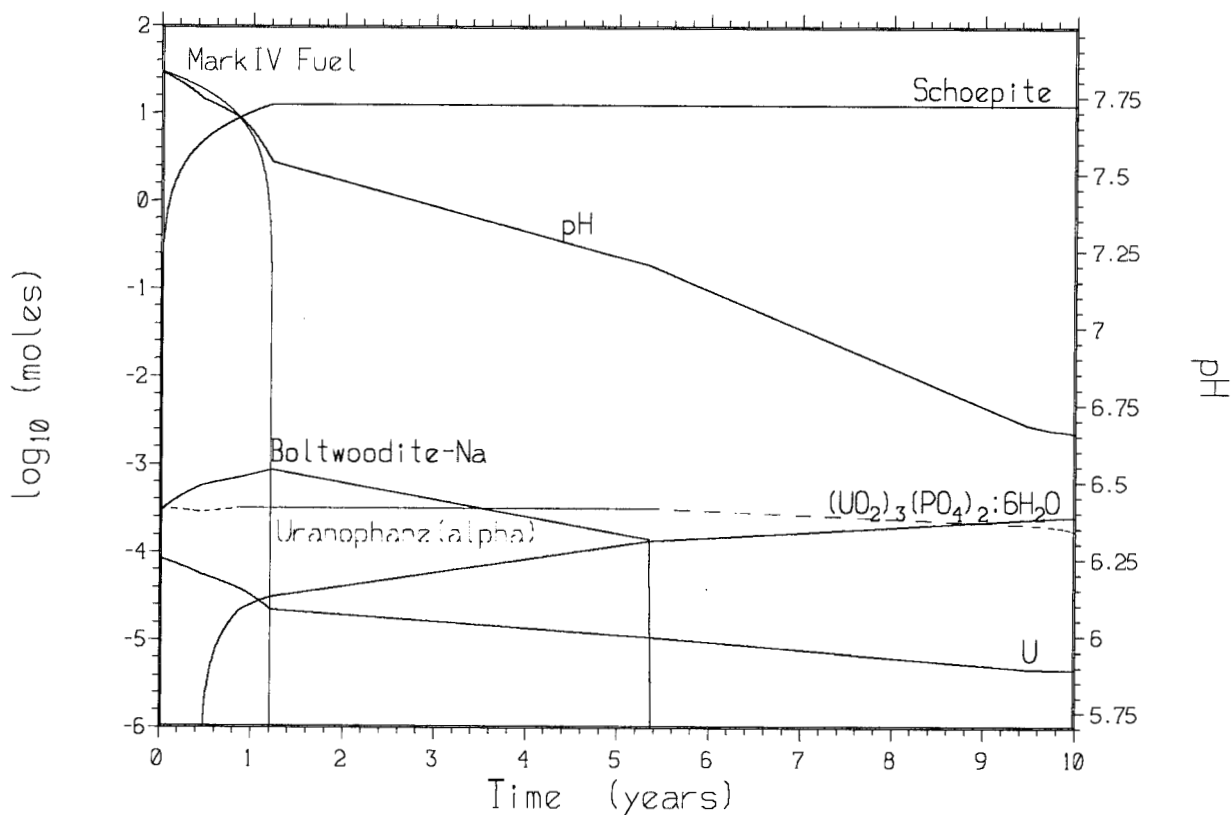


Figure 8. Predicted Concentration of N-Reactor SNF, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time Early in Case 3 (n40h1112)

Table 15. Predicted Elemental Composition of Corrosion Products (kg), Total Mass (kg) and Density in Selected Years for Case 4 (n10h1111)

Years	1656	122610	634310
Element			
O	3.865E+03	6.604E+03	1.124E+04
Al	1.964E-01	1.337E+01	6.475E+01
B	2.522E-15	9.388E-13	0.000E+00
Ba	1.233E-02	8.471E-01	4.119E+00
Ca	9.560E-02	5.263E+00	2.752E+01
Cl	0.000E+00	0.000E+00	1.603E-13
Cr	0.000E+00	3.207E-01	1.559E+00
Cu	2.122E+02	2.036E+02	1.927E+02
F	0.000E+00	0.000E+00	3.861E-13
Fe	1.310E+03	6.801E+03	1.491E+04
H	1.628E+02	1.659E+02	1.782E+02
C	0.000E+00	0.000E+00	9.231E-12
P	5.001E-01	4.237E+00	1.002E+01
K	6.418E-03	3.749E+00	1.975E+01
Li	0.000E+00	0.000E+00	1.328E-15
Mg	6.780E-02	4.353E+00	2.144E+01
Mn	1.591E+01	1.778E+02	4.185E+02
Mo	1.551E-01	1.260E+01	6.225E+01
N	0.000E+00	0.000E+00	7.225E-14
Na	0.000E+00	9.591E-01	7.510E+00
Ni	0.000E+00	3.437E+01	4.017E+02
S	9.257E-02	0.000E+00	1.449E-11
Si	6.833E+00	2.017E+02	8.174E+02
U	9.609E+03	9.619E+03	9.658E+03
Total (kg)	1.518E+04	2.385E+04	3.804E+04
Density (g/cm ³)	4.936	4.947	4.855

Table 16. Predicted Solution Elemental Composition (mole kg⁻¹) and pH in Selected Years for Case 4 (n10h1111)

Years	1656	122610	634310
pH	5.53	5.87	6.68
Element			
Al	1.866E-10	1.177E-11	2.087E-14
B	4.502E-03	8.359E-03	9.421E-03
Ba	5.060E-07	5.961E-07	3.388E-07
Ca	1.555E-04	6.426E-05	9.161E-05
Cl	2.014E-04	2.014E-04	2.014E-04
Cr	1.015E-01	5.382E-02	7.020E-03
Cu	1.180E-03	2.338E-04	3.725E-06
F	1.410E-04	1.635E-04	1.697E-04
Fe	9.919E-12	5.720E-12	1.690E-12
C	3.175E-05	4.781E-05	1.199E-04
P	1.676E-08	2.906E-08	1.282E-07
K	1.250E-03	1.776E-03	1.173E-03
Li	3.131E-06	6.915E-06	6.915E-06
Mg	7.807E-05	1.265E-04	5.251E-05
Mn	1.606E-10	3.231E-11	4.852E-13
Mo	1.398E-03	3.007E-03	8.164E-04
N	2.173E-03	1.301E-03	2.795E-04
Na	1.088E-02	1.838E-02	1.236E-02
Ni	5.070E-02	2.423E-02	3.622E-04
S	2.186E-03	4.582E-04	3.389E-04
Si	1.511E-04	1.856E-04	1.859E-04
U	1.808E-05	9.097E-06	8.810E-06

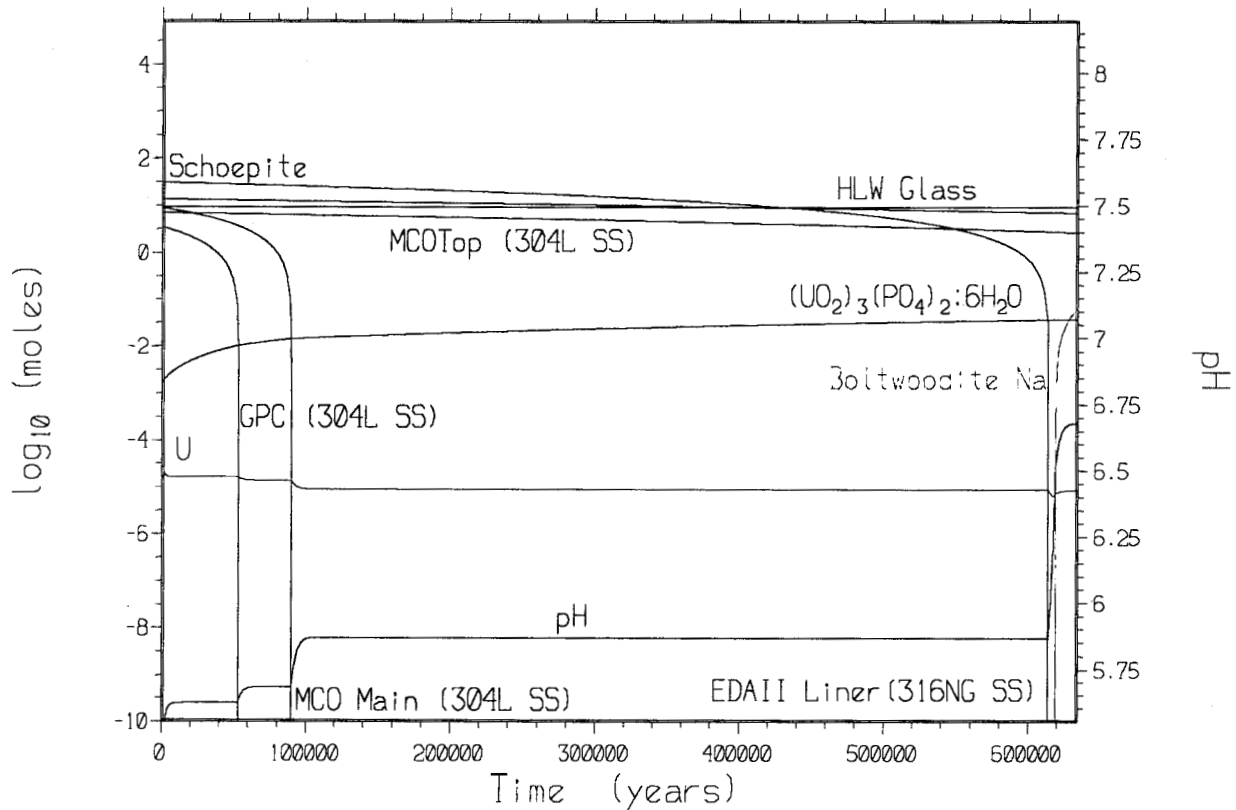


Figure 9. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time During Case 4 (n10H1111)

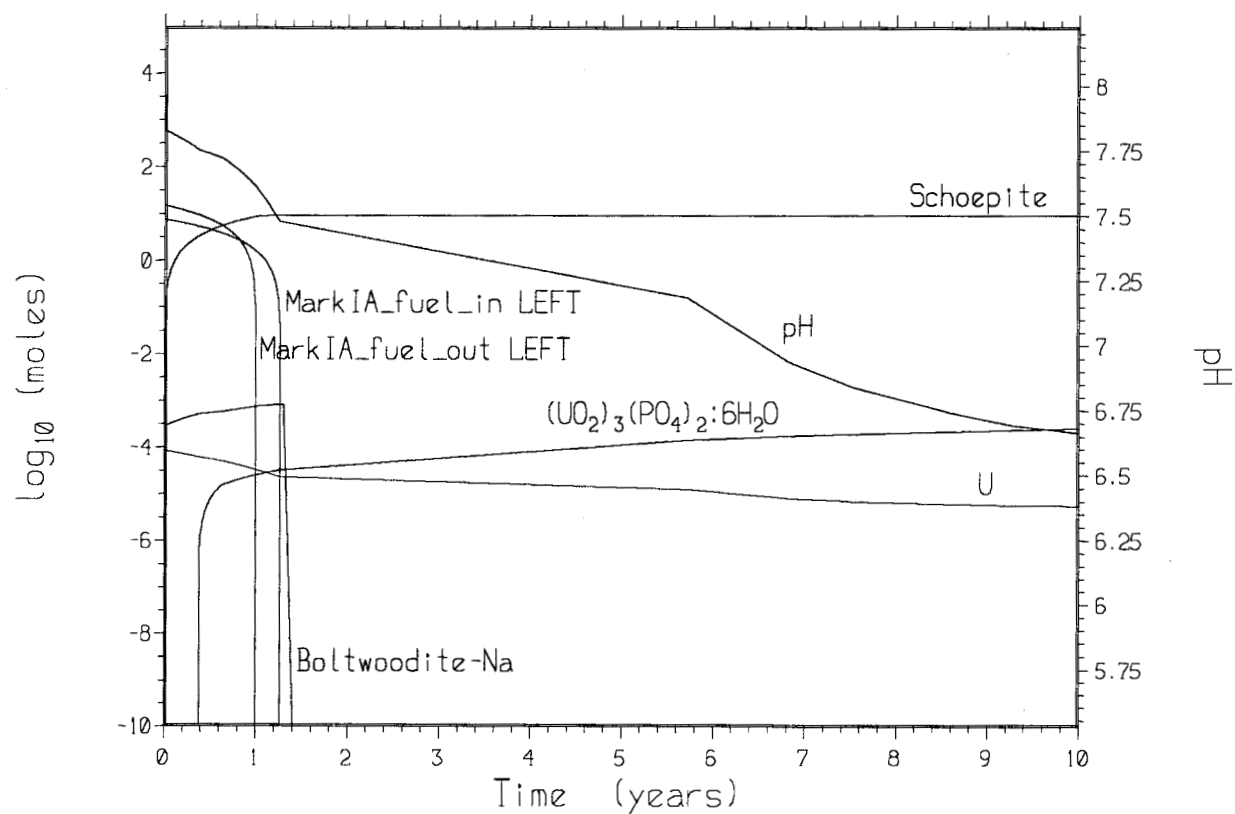


Figure 10. Predicted Concentration of N-Reactor SNF, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time Early in Case 4 (n10H1111)

Table 17. Predicted Elemental Composition of Corrosion Products (kg), Total Mass (kg) and Density in Selected Years for Case 5 (n10h2111)

Years	155	49916	634320
Element			
O	4.960E+03	1.056E+04	1.274E+04
Al	9.041E-02	8.418E+00	1.342E+02
B	3.403E-16	7.083E-12	5.473E-19
Ba	5.044E-03	5.343E-01	8.696E+00
Ca	6.831E-02	3.241E+00	4.945E+01
Cr	2.981E-07	0.000E+00	0.000E+00
Cu	6.540E+00	1.523E+02	1.523E+02
F	0.000E+00	0.000E+00	2.182E+00
Fe	3.888E+03	1.546E+04	1.598E+04
H	1.580E+02	1.647E+02	1.862E+02
C	1.367E-12	0.000E+00	7.605E-01
P	2.206E+00	1.013E+01	1.067E+01
K	7.238E-17	2.708E+00	3.627E+01
Mg	1.125E-17	2.255E+00	4.128E+01
Mn	9.175E+01	4.415E+02	4.415E+02
Mo	1.635E-01	5.013E+00	0.000E+00
Na	1.702E-16	5.446E+00	7.218E+02
Ni	0.000E+00	9.762E+01	9.753E+01
S	1.101E+00	1.247E-01	1.113E-17
Si	3.430E+01	2.780E+02	1.480E+03
U	9.602E+03	9.614E+03	9.620E+03
Total (kg)	1.874E+04	3.681E+04	4.170E+04
Density (g/cm ³)	4.993	5.030	4.674

Table 18. Predicted Solution Elemental Composition (mole kg⁻¹) and pH in Selected Years for Case 5 (n10h2111)

Years	155	49916	634320
pH	3.81	7.03	8.21
Element			
Al	1.348E-06	5.592E-15	1.038E-07
B	2.612E-03	1.032E-02	1.239E-05
Ba	1.372E-06	3.531E-07	3.687E-09
Ca	7.761E-05	3.554E-05	5.610E-06
Cl	2.014E-04	2.014E-04	2.014E-04
Cr	3.285E+00	5.612E-04	1.000E-16
Cu	1.327E-01	7.546E-07	7.478E-08
F	1.299E-04	1.749E-04	1.147E-04
Fe	3.371E-10	1.340E-12	1.138E-12
C	3.318E-06	2.182E-04	3.752E-03
P	1.342E-08	3.145E-07	1.156E-06
K	8.002E-04	7.440E-04	2.522E-03
Li	3.667E-07	6.915E-06	6.915E-06
Mg	3.800E-04	2.091E-05	2.362E-04
Mn	1.135E-07	8.439E-14	5.594E-16
Mo	3.566E-03	1.569E-03	1.000E-16
N	6.444E-02	1.538E-04	1.416E-04
Na	7.141E-03	5.292E-03	1.829E-03
Ni	1.540E+00	6.199E-05	2.601E-07
S	3.764E-03	6.730E-04	1.915E-04
Si	9.532E-05	1.862E-04	3.689E-05
U	6.891E-03	1.446E-05	5.085E-04

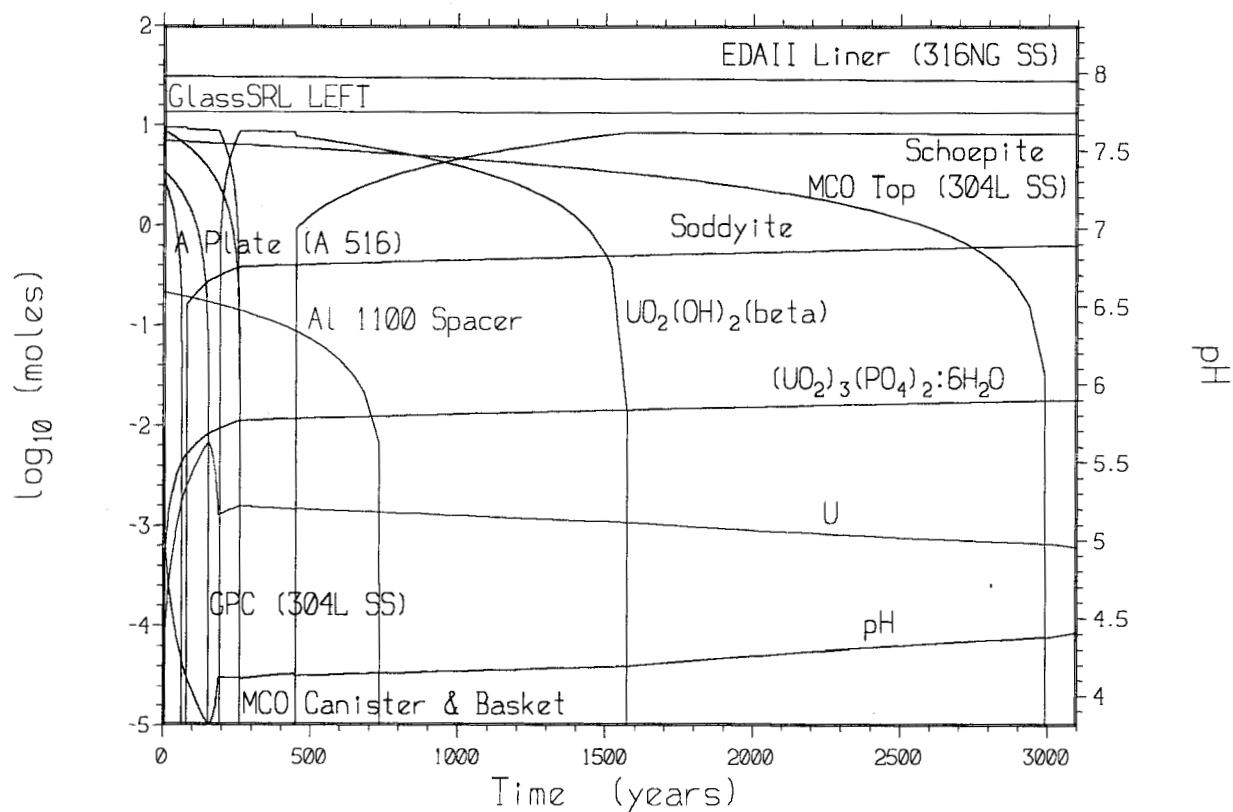


Figure 11. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time Early in Case 5 (n10H2111)

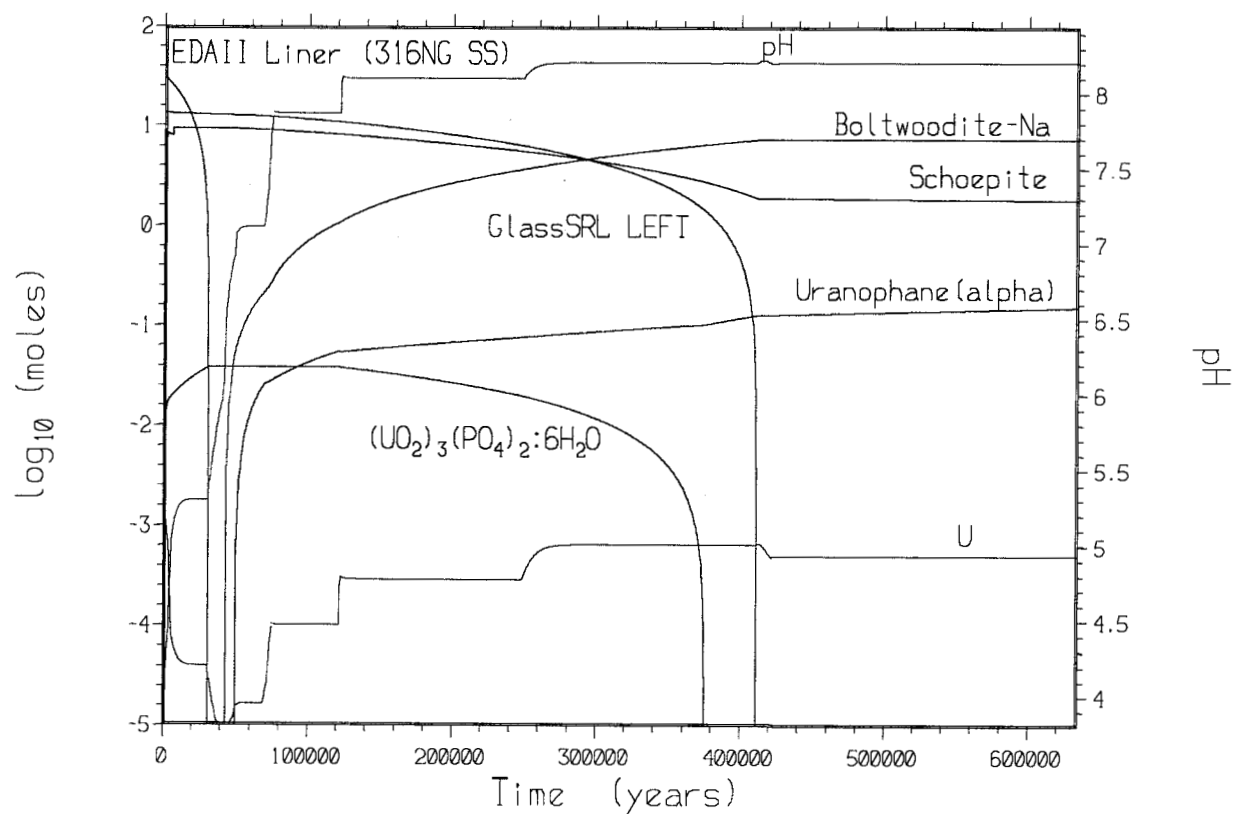


Figure 12. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time During Case 5 (n10H211)

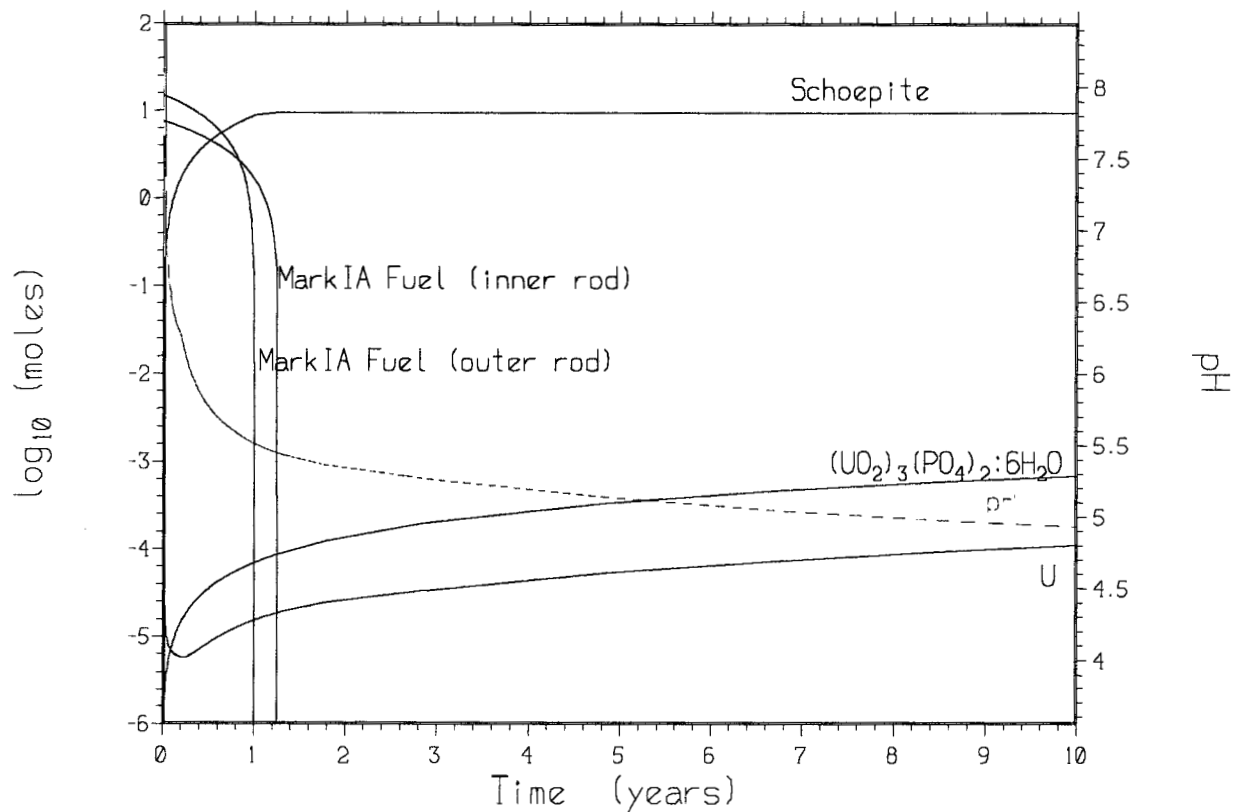


Figure 13. Predicted Concentration of N-Reactor SNF, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time Early in Case 5 (n10H2111)

Table 19. Predicted Elemental Composition of Corrosion Products (kg), Total Mass (kg) and Density in Selected Years for Case 6 (n10h1112, n11h1112, n12h1112)

Years	317	102350	633820
Element			
O	3.791E+03	6.423E+03	1.145E+04
Al	3.686E-02	1.019E+01	6.259E+01
B	3.357E-15	0.000E+00	0.000E+00
Ba	2.137E-03	5.179E-01	3.224E+00
Ca	2.638E-02	6.519E+00	5.679E+01
Cr	0.000E+00	1.961E-01	2.406E-17
Cu	8.872E+01	1.978E+02	1.872E+02
Fe	1.224E+03	6.464E+03	1.490E+04
H	1.628E+02	1.651E+02	1.794E+02
C	0.000E+00	0.000E+00	2.820E-01
P	4.423E-01	4.046E+00	1.033E+01
Mg	6.011E-04	1.101E+00	1.155E+01
Mn	1.338E+01	1.679E+02	4.185E+02
Mo	0.000E+00	1.839E+00	2.411E+01
Ni	0.000E+00	1.161E+01	4.926E+02
S	6.007E-02	0.000E+00	1.113E-17
Si	4.361E+00	1.985E+02	9.963E+02
U	9.609E+03	9.614E+03	9.643E+03
Total (kg)	1.489E+04	2.327E+04	3.843E+04
Density (g/cm³)	4.923	4.946	4.811

Table 20. Predicted Solution Elemental Composition (mole kg⁻¹) and pH in Selected Years for Case 6 (n10h1112, n11h1112, n12h1112)

Years	317	102350	633820
pH	5.57	6.27	7.06
Element			
Al	8.373E-11	1.830E-13	4.331E-15
B	6.882E-04	8.200E-04	1.231E-03
Ba	2.474E-07	6.285E-07	7.046E-07
Ca	2.718E-04	2.193E-04	3.762E-05
Cl	2.014E-04	2.014E-04	2.014E-04
Cr	1.567E-02	5.383E-03	7.037E-04
Cu	6.518E-04	2.167E-05	6.131E-07
F	1.187E-04	1.195E-04	1.219E-04
Fe	8.496E-12	2.617E-12	1.317E-12
C	3.634E-05	6.662E-05	2.324E-04
P	1.814E-08	5.037E-08	3.448E-07
K	3.034E-04	3.375E-04	4.435E-04
Li	4.835E-06	6.915E-06	6.915E-06
Mg	1.542E-04	1.266E-04	2.200E-05
Mn	8.606E-11	2.940E-12	6.802E-14
Mo	2.609E-04	2.944E-04	1.347E-03
N	4.554E-04	2.576E-04	1.554E-04
Na	3.335E-03	3.592E-03	4.405E-03
Ni	7.832E-03	2.199E-03	5.077E-05
S	1.581E-03	2.178E-04	2.101E-04
Si	1.666E-04	1.857E-04	1.863E-04
U	1.491E-05	6.349E-06	1.529E-05

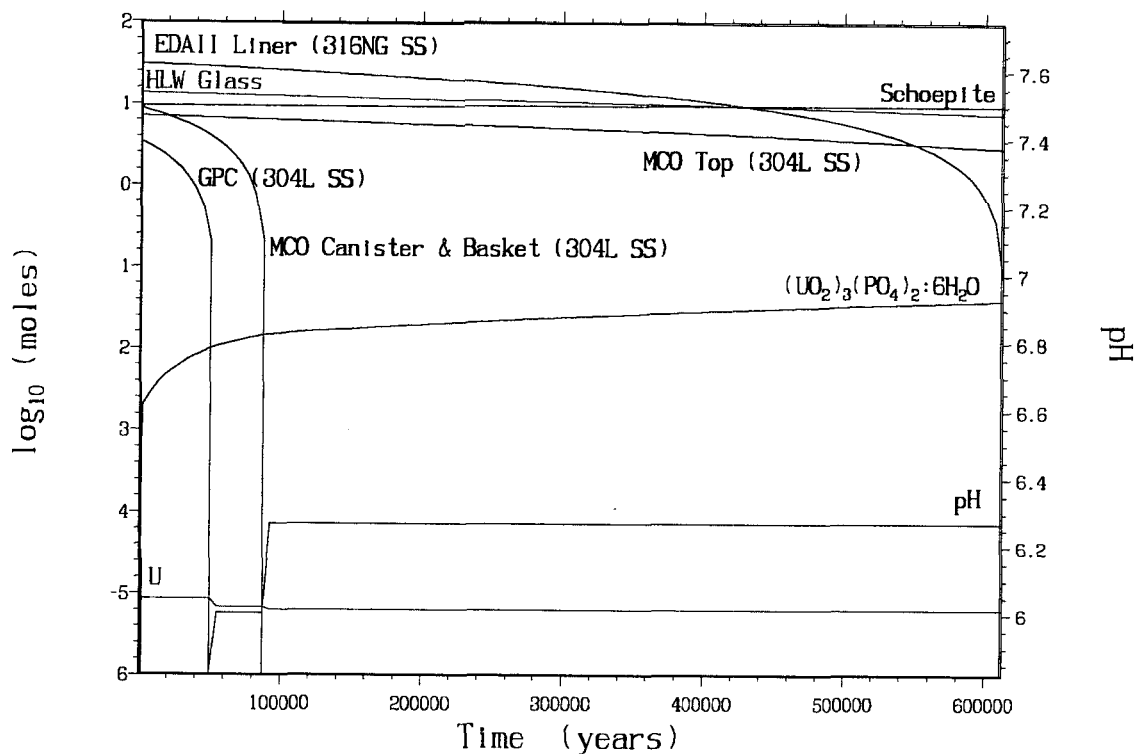


Figure 14. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time During Case 6 (n10H1112)

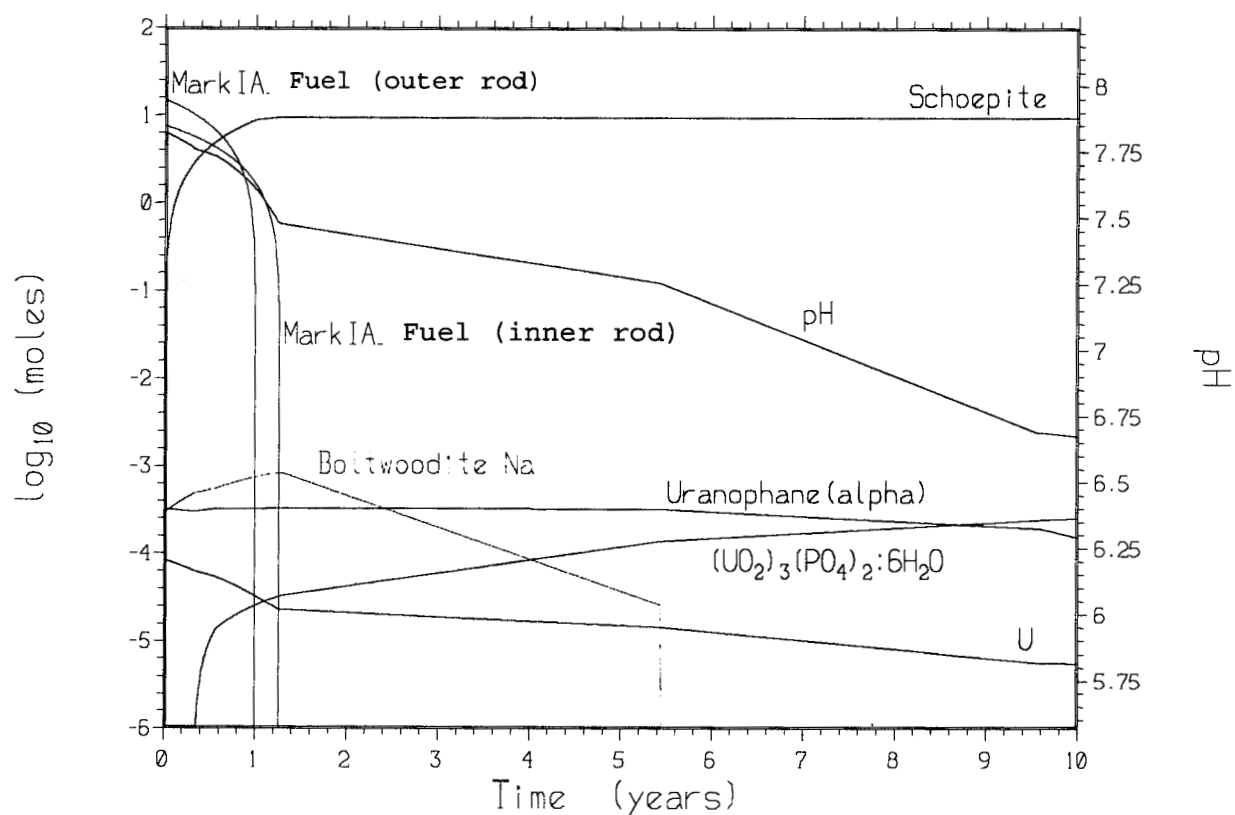


Figure 15. Predicted Concentration of N-Reactor SNF, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time Early in Case 6 (n10H1112)

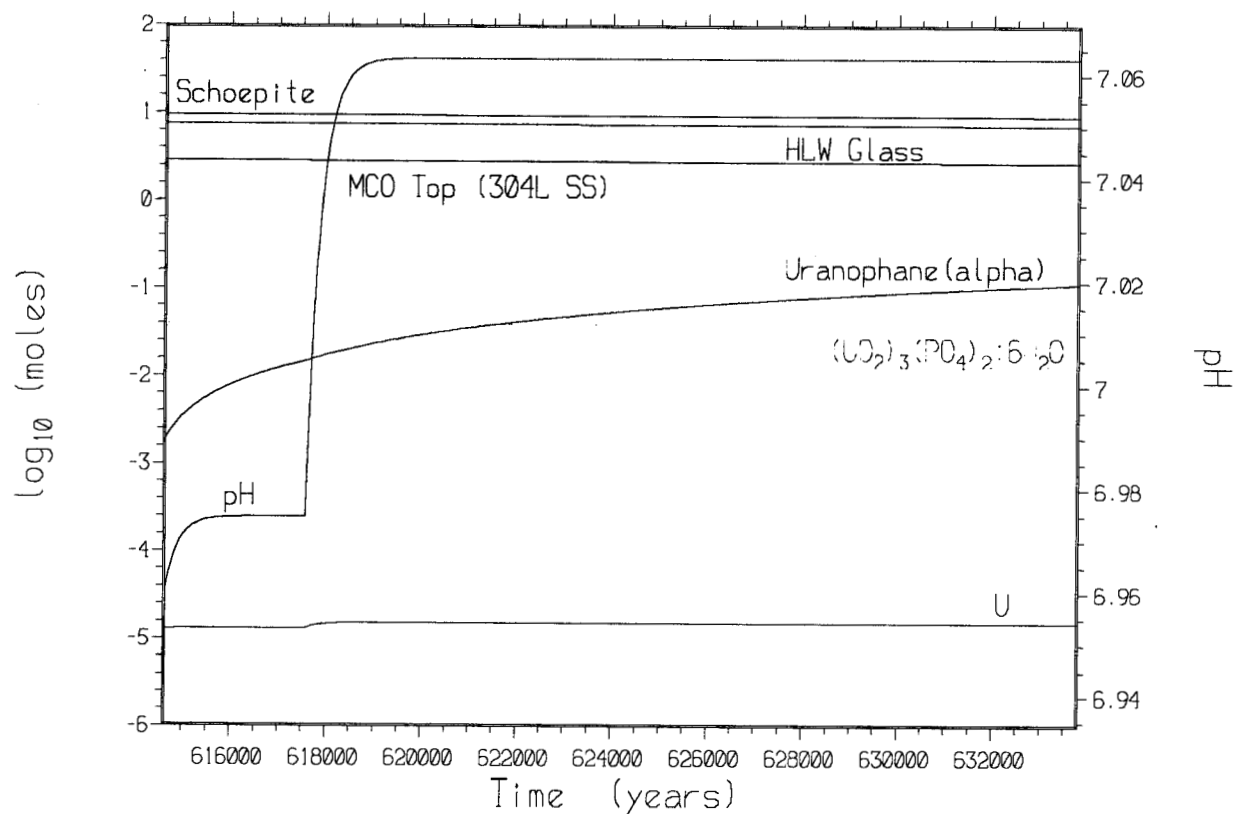


Figure 16. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time at the End of Case 6 (n12H1112)

Table 21. Predicted Elemental Composition of Corrosion Products (kg), Total Mass (kg) and Density in Selected Years for Case 7 (n10#1111)

Years	1656	301830	633810
Element			
O	3.865E+03	8.274E+03	1.124E+04
Al	1.963E-01	3.131E+01	6.470E+01
B	2.522E-15	0.000E+00	0.000E+00
Ba	1.233E-02	1.989E+00	4.115E+00
Ca	9.560E-02	1.307E+01	2.750E+01
Cl	0.000E+00	0.000E+00	1.953E-13
Cr	0.000E+00	7.531E-01	1.558E+00
Cu	2.122E+02	1.996E+02	1.927E+02
F	0.000E+00	2.198E-18	4.707E-13
Fe	1.310E+03	9.741E+03	1.491E+04
H	1.628E+02	1.702E+02	1.782E+02
C	2.758E-12	5.776E-12	6.424E-12
P	5.001E-01	6.332E+00	1.002E+01
K	6.461E-03	9.094E+00	1.971E+01
Li	0.000E+00	0.000E+00	1.618E-15
Mg	6.778E-02	1.030E+01	2.142E+01
Mn	1.591E+01	2.652E+02	4.185E+02
Mo	1.553E-01	3.128E+01	6.231E+01
N	0.000E+00	3.888E-17	8.810E-14
Na	0.000E+00	1.848E+00	7.366E+00
Ni	0.000E+00	1.658E+02	4.016E+02
S	9.282E-02	0.000E+00	1.766E-11
Si	6.832E+00	4.184E+02	8.169E+02
U	9.609E+03	9.633E+03	9.658E+03
Total (kg)	1.518E+04	2.897E+04	3.803E+04
Density (g/cm ³)	4.936	4.905	4.855

Table 22. Predicted Solution Elemental Composition (mole kg⁻¹) and pH in Selected Years for Case 7
(n10#1111)

Years	1656	301830	633810
pH	5.53	5.87	6.68
Element			
Al	1.867E-10	1.177E-11	2.088E-14
B	4.501E-03	8.359E-03	9.417E-03
Ba	5.060E-07	5.961E-07	3.387E-07
Ca	1.555E-04	6.426E-05	9.167E-05
Cl	2.014E-04	2.014E-04	2.014E-04
Cr	1.015E-01	5.382E-02	7.023E-03
Cu	1.180E-03	2.338E-04	3.728E-06
F	1.410E-04	1.635E-04	1.697E-04
Fe	9.917E-12	5.720E-12	1.691E-12
C	3.173E-05	4.781E-05	1.199E-04
P	1.675E-08	2.906E-08	1.282E-07
K	1.249E-03	1.776E-03	1.173E-03
Li	3.131E-06	6.915E-06	6.915E-06
Mg	7.804E-05	1.265E-04	5.255E-05
Mn	1.605E-10	3.231E-11	4.855E-13
Mo	1.398E-03	3.007E-03	8.160E-04
N	2.173E-03	1.301E-03	2.796E-04
Na	1.088E-02	1.838E-02	1.236E-02
Ni	5.070E-02	2.423E-02	3.625E-04
S	2.185E-03	4.582E-04	3.388E-04
Si	1.510E-04	1.856E-04	1.859E-04
U	1.808E-05	9.097E-06	8.808E-06

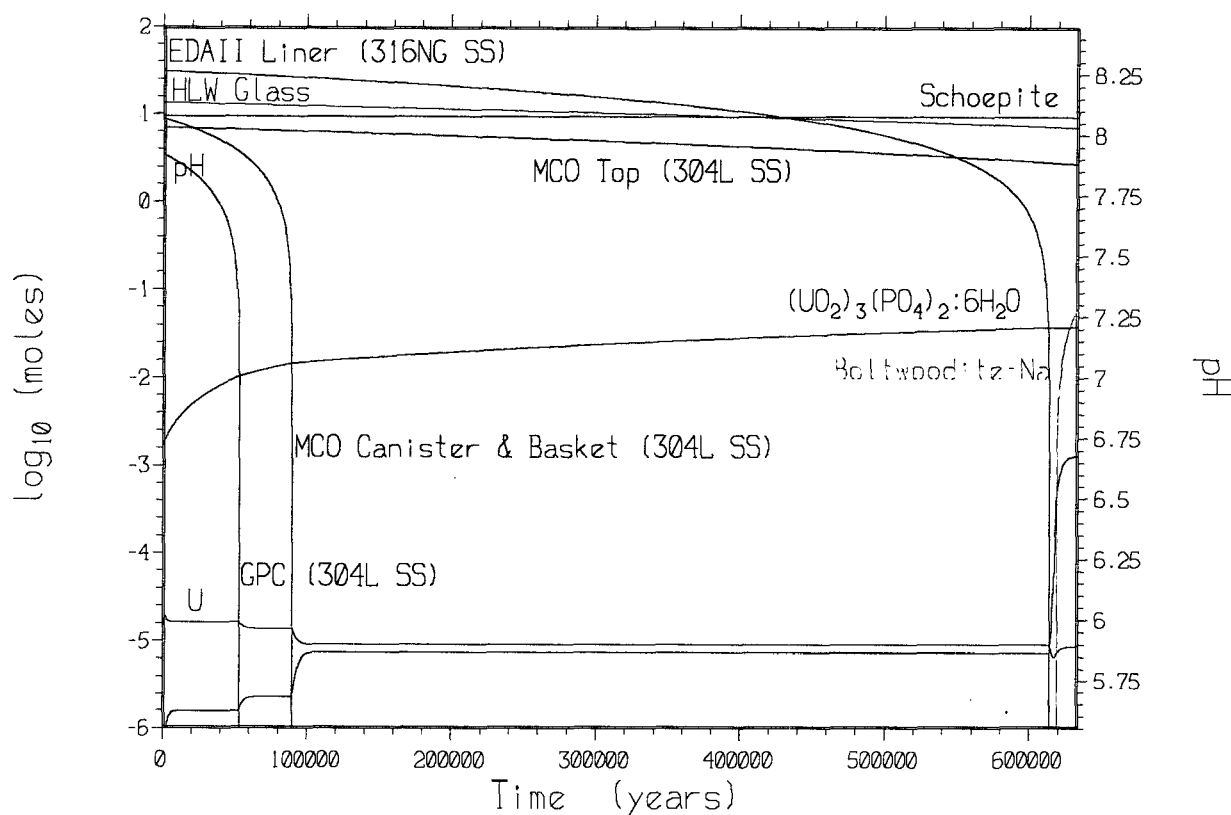


Figure 17. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time During Case 7 (n10#1111)

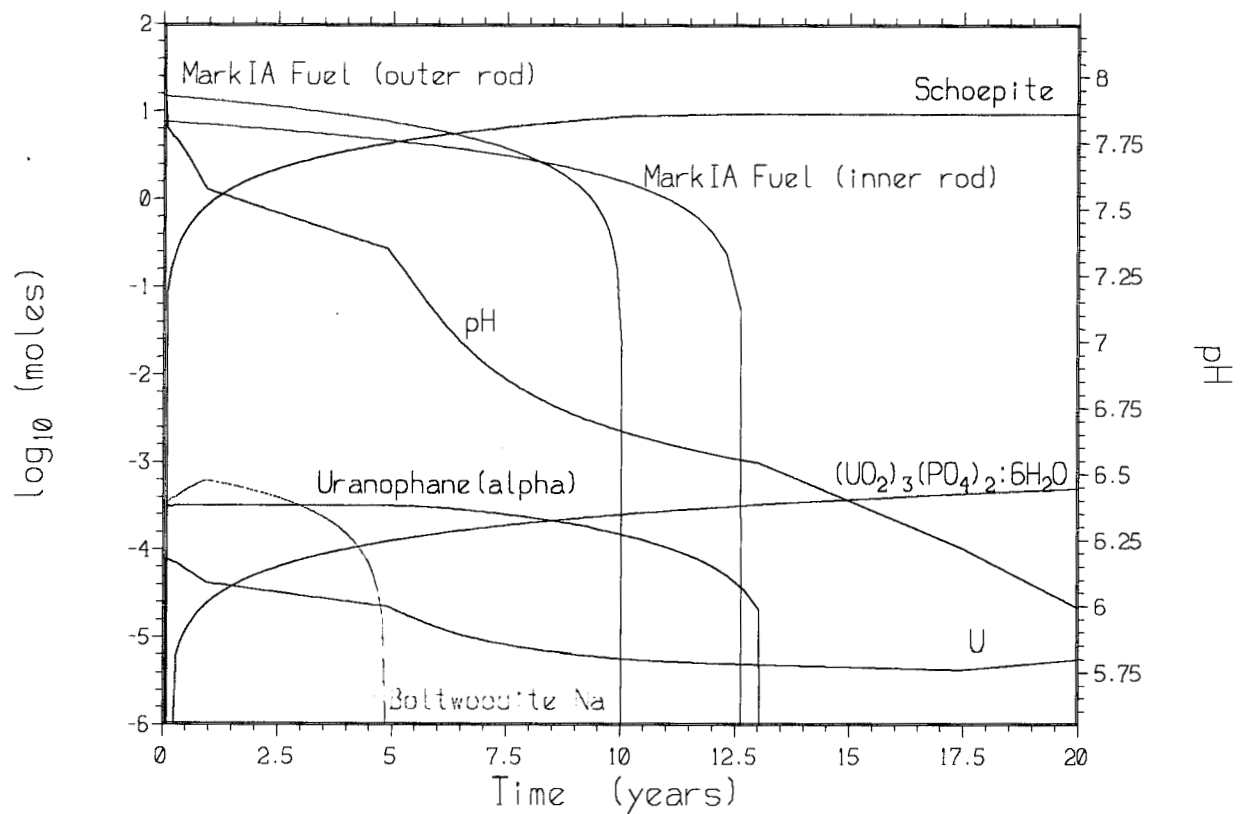


Figure 18. Predicted Concentration of N-Reactor SNF, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time Early in Case 7 (n10#1111)

Table 23. Predicted Elemental Composition of Corrosion Products (kg), Total Mass (kg) and Density in Selected Years for Case 8 (n10&1111, n11&1111)

Years	2014	105440	634090
Element			
O	6.460E+02	3.219E+03	8.023E+03
Al	2.430E-01	1.188E+01	6.614E+01
B	0.000E+00	0.000E+00	1.880E-13
Ba	1.521E-02	7.507E-01	4.208E+00
Ca	1.021E-01	4.564E+00	2.796E+01
Cl	3.365E-13	1.912E-13	0.000E+00
Cr	1.157E-07	2.842E-01	0.000E+00
Cu	2.121E+02	2.024E+02	1.910E+02
F	7.146E-12	3.961E-12	1.129E-01
Fe	1.333E+03	6.520E+03	1.492E+04
H	5.829E-02	2.848E+00	1.590E+01
C	1.676E-12	7.680E-12	3.680E-01
P	5.110E-01	3.937E+00	6.797E+00
K	4.832E-12	3.303E+00	1.976E+01
Li	1.548E-15	5.915E-16	0.000E+00
Mg	8.066E-02	3.838E+00	2.165E+01
Mn	1.658E+01	1.694E+02	4.185E+02
Mo	1.306E-01	1.092E+01	5.440E+01
N	1.995E-11	4.812E-12	0.000E+00
Na	1.544E-11	9.183E-01	4.766E+00
Ni	1.039E-10	2.794E+01	4.076E+02
S	3.551E-03	1.111E-11	0.000E+00
Si	7.547E+00	1.830E+02	8.304E+02
U	1.933E-01	9.461E+00	5.268E+01
Total (kg)	2.216E+03	1.038E+04	2.506E+04
Density (g/cm ³)	5.349	5.063	4.845

Table 24. Predicted Solution Elemental Composition (mole kg⁻¹) and pH in Selected Years for Case 8 (n10&1111, n11&1111)

Years	2014	105440	634090
pH	5.52	5.87	7.88
Element			
Al	2.020E-10	1.246E-11	1.020E-15
B	5.305E-03	8.375E-03	1.958E-02
Ba	6.354E-07	5.969E-07	2.626E-08
Ca	1.847E-04	6.447E-05	2.323E-05
Cl	2.014E-04	2.014E-04	2.014E-04
Cr	1.178E-01	5.408E-02	7.294E-03
Cu	1.474E-03	2.358E-04	1.764E-07
F	1.459E-04	1.636E-04	1.096E-14
Fe	1.251E-11	5.777E-12	1.139E-12
C	3.857E-05	4.714E-05	1.477E-03
P	2.042E-05	2.475E-05	2.606E-03
K	1.496E-03	1.779E-03	3.083E-03
Li	3.633E-06	6.915E-06	6.915E-06
Mg	9.312E-05	1.270E-04	5.239E-04
Mn	2.005E-10	3.248E-11	2.522E-15
Mo	1.715E-03	3.006E-03	6.636E-03
N	2.499E-03	1.306E-03	2.796E-04
Na	1.247E-02	1.841E-02	3.241E-02
Ni	5.882E-02	2.436E-02	1.801E-06
S	2.521E-03	4.590E-04	4.783E-04
Si	1.855E-04	1.855E-04	1.961E-04
U	3.147E-07	2.762E-07	6.337E-06

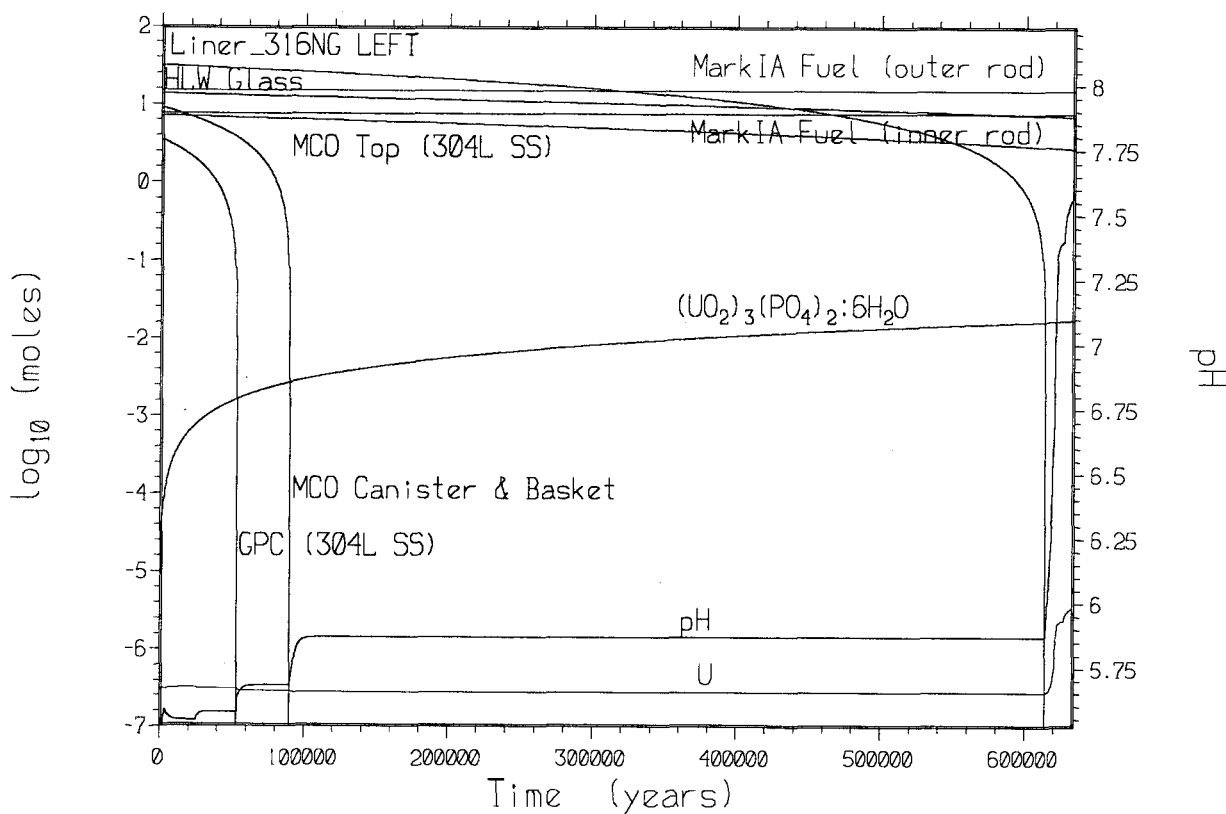


Figure 19. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time During Case 8 (n10&1111)

Case 9 predicted the corrosion of Mark IA SNF (assuming a decayed, instead of fresh, fuel composition in the WP) with low steel and high HLW glass degradation rates and low drip rates. Figure 20 shows the predicted concentration of major WP components, precipitated Np, Pu, and U Minerals, total Np, Pu, and U in solution, and pH as a function of time during Case 9 (n10h1211). The predicted soluble U concentration is fairly constant and is controlled mainly by the formation of schoepite ($\text{UO}_3 \cdot 2\text{H}_2\text{O}$) with smaller amounts of $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 6\text{H}_2\text{O}$ also precipitating. Sodium boltwoodite ($\text{NaUO}_2\text{SiO}_3\text{OH} \cdot 1.5\text{H}_2\text{O}$) is an important sink for U during the period from $\sim 90,000$ to $200,000$ years, during and following the predicted complete dissolution of the HLW glass. Soluble Pu from SNF degradation is held to low levels by formation of PuO_2 . The predicted soluble Np concentrations are similar to those of U from $\sim 200,000$ to $600,000$ years. However, only a small amount of NpO_2 is predicted to form at the beginning of this period and the predicted amount of NpO_2 decreases through 6.35×10^5 years leading to large predicted losses of Np from the WP (Table 8). There is more than a five-fold decrease in the predicted kg of Np in the WP corrosion products between $\sim 1.35 \times 10^5$ and $\sim 6.35 \times 10^5$ years (shown in Table 25).

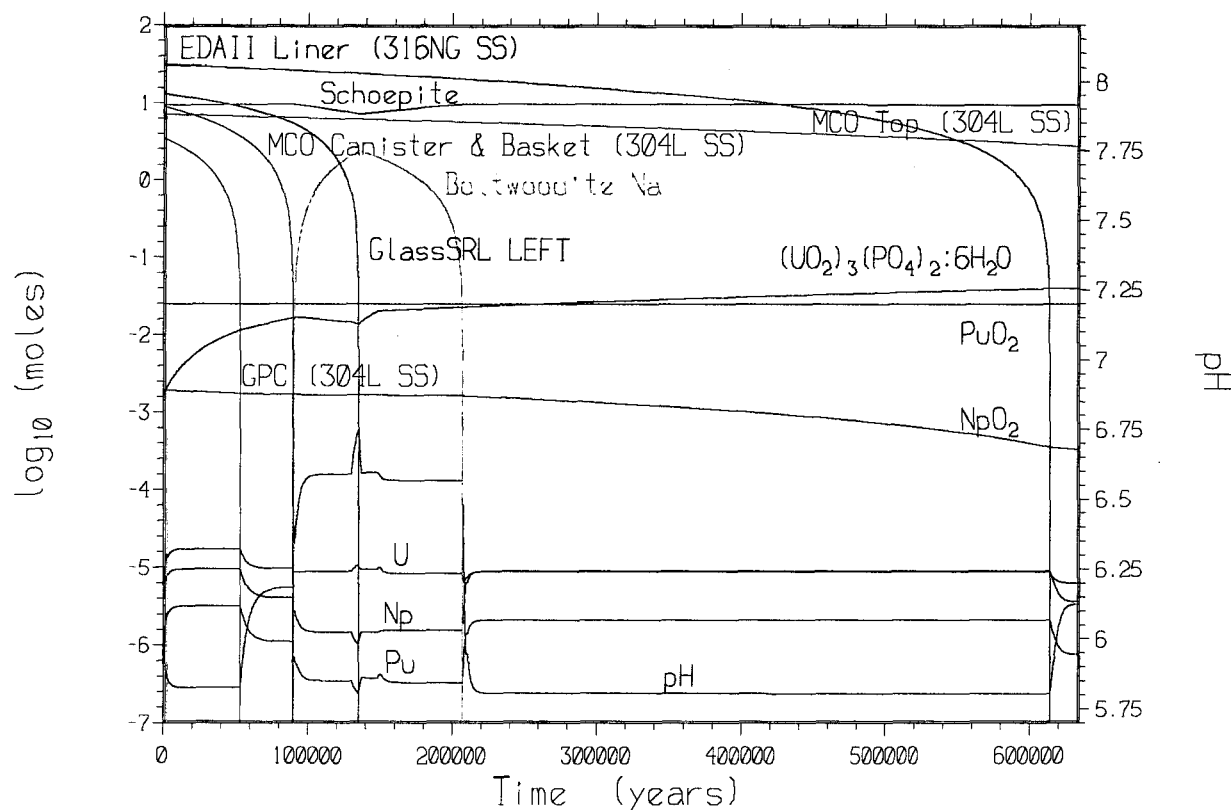


Figure 20. Predicted Concentration of Major Waste Package Components, Precipitated Np, Pu, and U Minerals, Total Np, Pu, and U in Solution, and pH as a Function of Time During Case 9 (n10h1211)

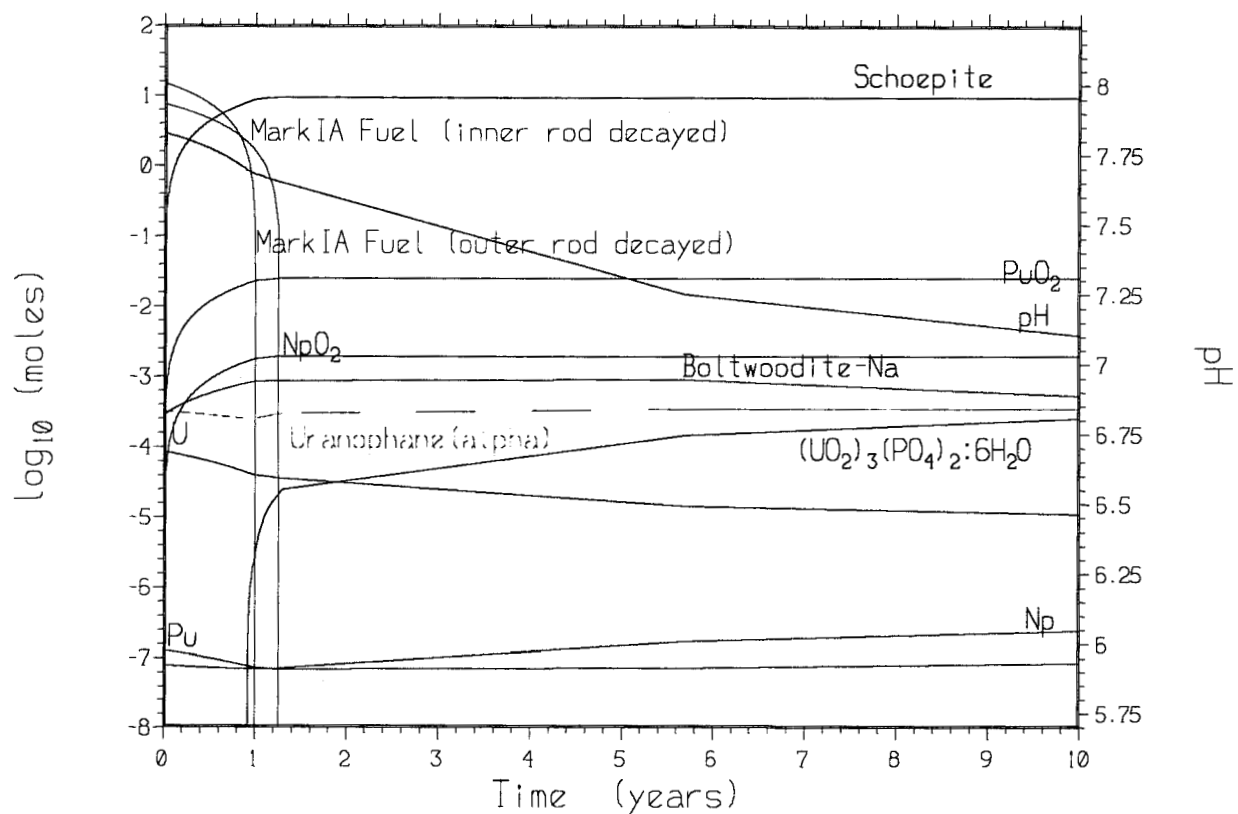


Figure 21. Predicted Concentration of N Reactor SNF, Precipitated Np, Pu, and U Minerals, Total Np, Pu, and U in Solution, and pH as a Function of Time Early in Case 9 (n10h1211)

Table 25. Predicted Elemental Composition of Corrosion Products (kg), Total Mass (kg) and Density in Selected Years for Case 9 (n10h1211)

Years	51305	134790	634080
Element			
O	6.107E+03	8.821E+03	1.238E+04
Al	4.529E+01	1.342E+02	1.341E+02
B	0.000E+00	3.845E-12	0.000E+00
Ba	2.937E+00	8.700E+00	8.635E+00
Ca	1.029E+01	3.120E+01	3.960E+01
Cl	2.317E-14	0.000E+00	1.635E-13
Cr	1.112E+00	3.294E+00	3.269E+00
Cu	2.107E+02	2.103E+02	1.977E+02
F	2.087E-12	3.239E-01	7.244E-12
Fe	4.652E+03	7.495E+03	1.520E+04
H	1.718E+02	1.887E+02	1.925E+02
C	0.000E+00	0.000E+00	7.690E-12
P	2.910E+00	5.175E+00	1.048E+01
K	1.676E+01	7.618E+01	3.901E+01
Li	4.131E-17	0.000E+00	2.116E-15
Mg	2.479E+00	1.737E+01	1.378E+01
Mn	1.095E+02	1.838E+02	4.185E+02
Mo	2.462E+01	6.650E+01	9.903E-14
N	6.692E-13	0.000E+00	1.470E-12
Na	2.003E+01	2.316E+02	1.143E+01
Ni	3.133E+02	6.845E+02	8.590E+02
Np	1.776E+00	1.692E+00	3.320E-01
Pu	2.543E+01	2.541E+01	2.509E+01
S	7.051E-12	0.000E+00	4.221E-12
Si	4.697E+02	1.338E+03	1.472E+03
U	9.569E+03	9.640E+03	9.638E+03
Total (kg)	2.176E+04	2.916E+04	4.065E+04
Density (g/cm ³)	4.764	4.526	4.708

Table 26. Predicted Solution Elemental Composition (mole kg⁻¹) and pH in Selected Years for Case 9 (n10h1211)

Years	51305	134790	634080
pH	5.83	6.75	6.12
Element			
Al	3.816E-05	4.060E-08	2.740E-07
B	7.363E-02	1.062E-01	1.239E-05
Ba	5.474E-07	1.434E-07	6.760E-07
Ca	1.078E-03	5.976E-04	5.997E-07
Cl	2.014E-04	2.014E-04	2.014E-04
Cr	2.239E-01	5.345E-02	7.056E-03
Cu	4.102E-04	4.535E-06	4.381E-05
F	5.445E-04	2.816E-04	1.148E-04
Fe	6.836E-12	1.685E-12	3.261E-12
C	4.691E-05	1.556E-04	5.738E-05
P	3.006E-08	2.678E-07	3.897E-08
K	1.357E-02	1.564E-02	9.573E-05
Li	6.915E-06	6.915E-06	6.915E-06
Mg	7.310E-03	1.015E-02	3.444E-07
Mn	5.658E-11	5.934E-13	5.987E-12
Mo	3.897E-04	3.632E-04	1.478E-06
N	4.629E-03	1.301E-03	2.795E-04
Na	1.364E-01	5.563E-02	1.010E-03
Ni	4.239E-02	4.416E-04	4.481E-03
Np	9.480E-06	1.026E-06	3.627E-06
Pu	3.186E-06	2.434E-07	7.638E-07
S	1.791E-03	1.802E-03	2.096E-04
Si	5.109E-05	4.133E-05	7.486E-05
U	1.732E-05	1.061E-05	6.320E-06

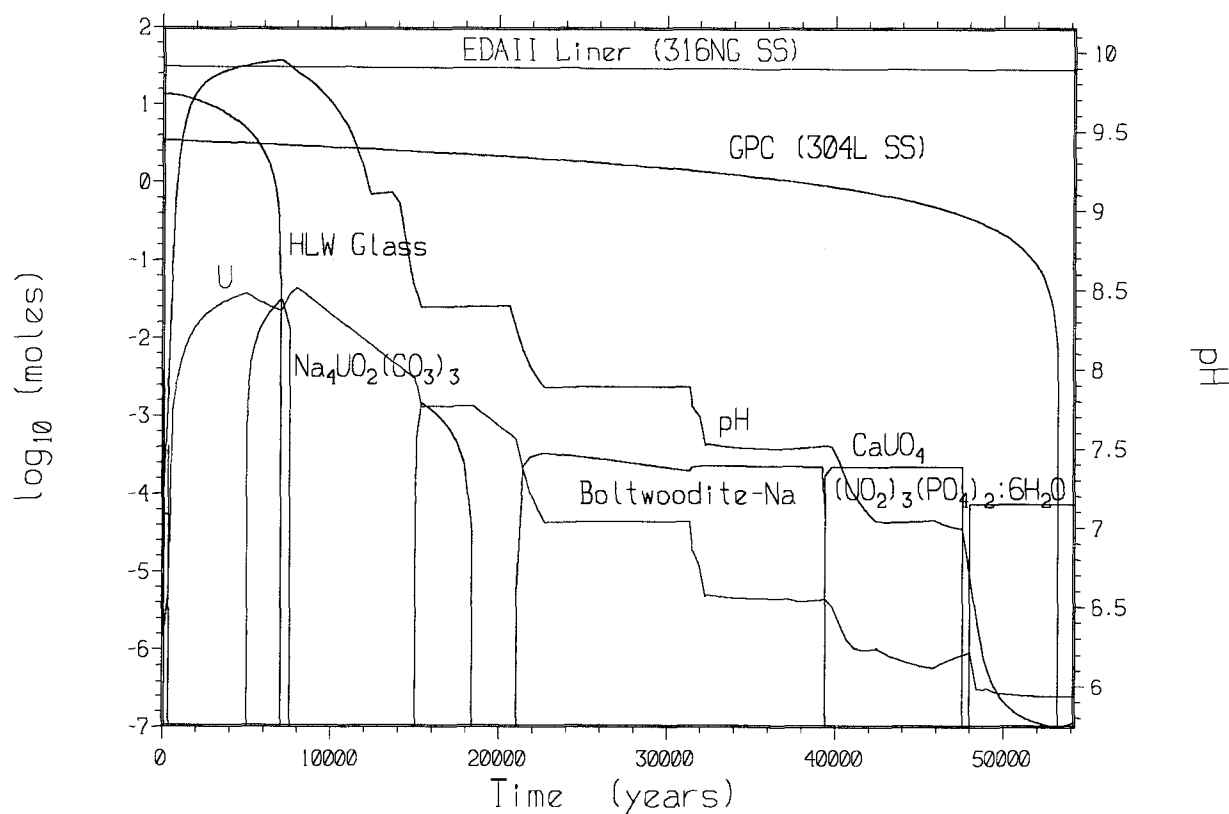


Figure 22. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time During the First-Stage of Case 10 (n10a1201)

Table 27. Predicted Mass Remaining of N-Reactor (U-metal) SNF WP Components in Selected Years for the First-stage of Case 10 (n10a1201)

Years	0	65	6953	54250
WP Component	Mass (kg)	Mass (kg)	Mass (kg)	Mass (kg)
HLW Glass	5766	5757	0	0
A-plate and MCO Stand (A516))	1224	0	0	0
EDAII Liner (316L SS)	13189	13188	13040	12024
GPC (304L SS))	1483	1481	1290	0

Source: "Nreactor_U.xls" sheet "Reactant Losses" (Attachment III)

Figure 22 shows the predicted partitioning of U between the WP solution and solid U-bearing phases during the degradation of HLW glass in the first stage of Case 10. The HLW glass degradation is complete before ~7000 years (Table 27) causing the WP solution to approach a

pH of 10 (Table 29). The predicted concentration of U in the WP solution remains high (10^{-3} to 10^{-1} molal) during the period of highest pH (~100 to 15,000 years). During the same time period, only small amounts of U-bearing solids are predicted to form (Figure 22 and Table 28) causing a predicted loss from the WP of most of the U from HLW glass degradation.

Degradation of stainless steel from the GPCs and the EDA II liner causes a predicted decrease in pH throughout the rest of the first stage (Figure 22, Table 27 and Table 29). The U in solution also decreases due to the formation of several U-bearing minerals.

Table 28. Predicted Elemental Composition of Corrosion Products (kg), Total Mass (kg) and Density in Selected Years for First Stage of Case 10 (n10a1201)

Years	65	6953	54251
Element			
O	5.332E+02	3.800E+03	3.474E+03
Al	2.045E-01	1.343E+02	1.343E+02
B	0.000E+00	1.541E+02	3.959E-13
Ba	1.320E-02	8.710E+00	8.708E+00
Ca	1.135E-01	3.762E+01	1.462E+01
Cl	1.102E-14	1.231E-17	0.000E+00
Cr	3.312E-11	0.000E+00	3.297E+00
F	1.076E-02	2.784E-01	4.522E-01
Fe	1.206E+03	1.987E+03	3.528E+03
H	4.902E-02	9.840E+01	2.722E+01
C	0.000E+00	3.253E+01	0.000E+00
P	1.857E-01	1.362E+00	2.307E+00
K	6.025E-02	9.912E+01	4.549E+01
Mg	7.047E-02	4.670E+01	3.269E+01
Mn	1.285E+01	1.964E+01	6.575E+01
Mo	1.593E-14	5.328E-16	1.282E+01
Na	5.516E-03	2.328E+02	0.000E+00
Ni	3.385E-01	3.726E+01	2.591E+02
S	3.083E-03	0.000E+00	0.000E+00
Si	5.592E+00	1.263E+03	1.285E+03
U	1.613E-01	3.324E+01	2.165E-01
Total (kg)	1.759E+03	7.985E+03	8.893E+03
Density (g/cm ³)	5.232	2.950	3.797

Table 29 shows that only ~0.2 kg of U is predicted to remain in the WP corrosion products by the end of the first stage of Case 10.

Table 29. Predicted Solution Elemental Composition (mole kg^{-1}) and pH in Selected Years for the First-stage of Case 10 (n10a1201)

Years	65	6953	54251
pH	7.11	9.95	5.77
Element			
Al	2.653E-15	2.090E-08	6.065E-09
B	5.930E-03	1.954E-01	9.944E-03
Ba	1.024E-07	2.582E-11	6.806E-07
Ca	1.012E-06	4.605E-06	2.089E-03
Cl	2.014E-04	2.014E-04	2.014E-04
Cr	2.570E-03	1.050E-01	9.430E-02
F	1.878E-05	8.715E-03	5.595E-08
Fe	1.313E-12	4.173E-12	7.278E-12
C	2.584E-04	1.192E+00	4.413E-05
P	1.833E-03	1.586E-04	1.997E-05
K	1.299E-03	1.707E-01	5.159E-03
Li	1.551E-07	6.339E-06	6.915E-06
Mg	8.938E-05	3.125E-06	1.062E-03
Mn	7.306E-14	2.026E-14	5.892E-11
Mo	8.369E-05	3.420E-03	1.260E-04
N	1.942E-04	2.292E-03	2.092E-03
Na	1.365E-02	2.245E+00	6.614E-03
Ni	2.856E-05	3.133E-10	4.424E-02
S	3.353E-03	2.367E-02	4.471E-04
Si	1.867E-04	5.290E-03	5.605E-05
U	1.616E-06	2.171E-02	2.521E-07

Figure 23 shows the predicted concentration of the major WP components, precipitated U minerals, total U in solution, and pH as a function of time during the second stage of Case 10 (n10b1011). Soluble U concentrations are kept at a fairly constant level by formation of schoepite ($\text{UO}_3 \cdot 2\text{H}_2\text{O}$) and $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 6\text{H}_2\text{O}$ (Figure 23 and Table 32). The amount of U in the WP corrosion products also remains nearly constant (Table 31). The pH of the WP solution is kept between 5.6 and 5.9 for most of the second stage of Case 10 by corrosion of 304L and 316L stainless steel WP components (Tables 30 and 32).

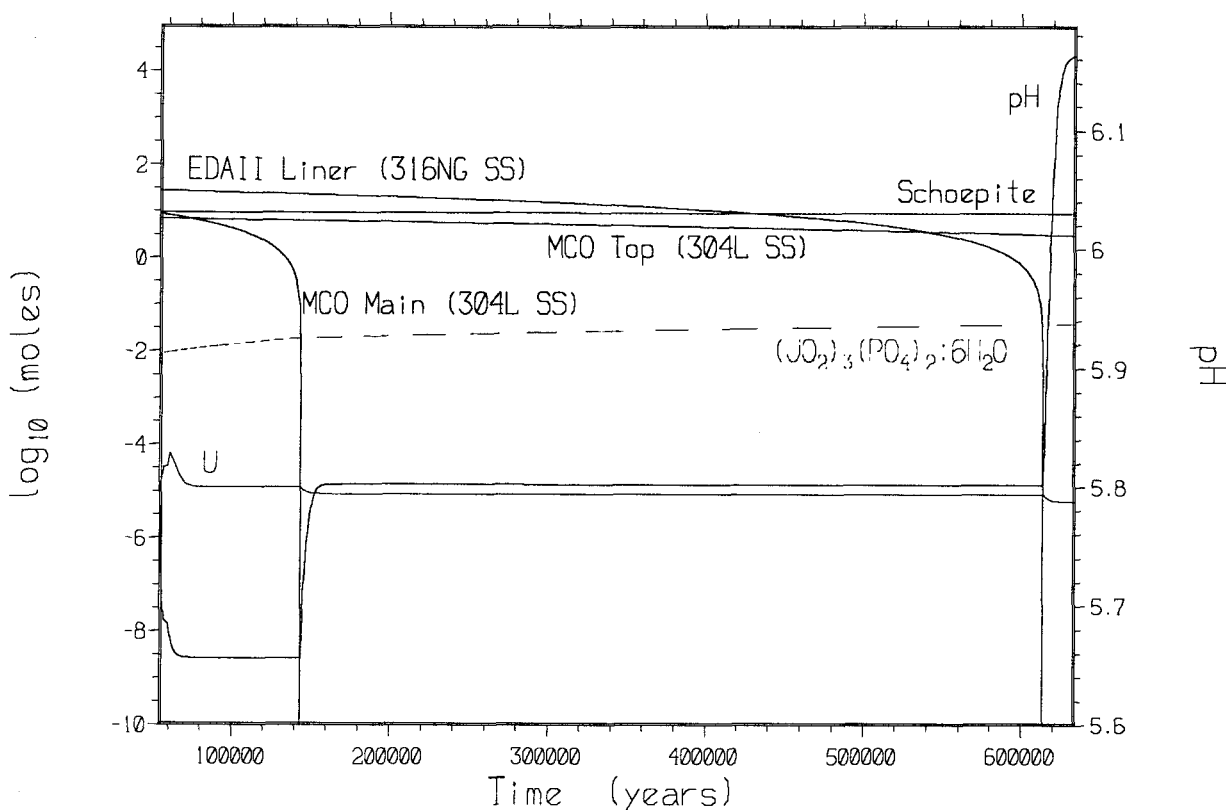


Figure 23. Predicted Concentration of Major Waste Package Components, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time During the Second-Stage of Case 10 (n10b1011)

Table 30. Predicted Mass Remaining of N-Reactor (U-metal) SNF WP Components in Selected Years for the Second-stage of Case 10 (n10b1011)

Years	54250	103210	350290	634080
WP Component	Mass (kg)	Mass (kg)	Mass (kg)	Mass (kg)
EDAll Liner (316L SS)	12024	10972	5665	0
MCO Main Canister and Basket (304L SS)	3781	1720	0	0
MCO Top (304L SS)	2986	2844	2130	1311
Mark IA Fuel (outer rod)	6382	0	0	0
Mark IA Fuel (inner rod)	3227	0	0	0
Al 1100 Spacer	91	0	0	0

Source: "Nreactor_U.xls" sheet "Reactant Losses" (Attachment III)

Table 31. Predicted Elemental Composition of Corrosion Products (kg), Total Mass (kg) and Density in Selected Years for the Second Stage of Case 10 (n10b1011)

Years	103210	350290	634080
Element			
O	7.757E+03	1.013E+04	1.213E+04
Al	1.341E+02	1.341E+02	1.341E+02
Ba	8.700E+00	8.662E+00	8.621E+00
Ca	1.356E+01	1.773E+01	2.313E+01
Cl	2.402E-14	0.000E+00	1.894E-13
Cr	3.294E+00	3.279E+00	3.264E+00
Cu	2.087E+02	1.991E+02	1.910E+02
F	8.393E-13	0.000E+00	6.565E-12
Fe	5.714E+03	1.083E+04	1.509E+04
H	1.897E+02	1.908E+02	1.919E+02
C	3.720E-12	0.000E+00	1.539E-11
P	3.777E+00	7.274E+00	1.021E+01
K	4.101E+01	3.262E+01	2.966E+01
Li	5.403E-17	2.008E-19	2.260E-15
Mg	3.225E+01	3.279E+01	3.360E+01
Mn	1.308E+02	2.856E+02	4.153E+02
Mo	1.189E+01	8.880E-16	9.737E-14
N	2.868E-12	0.000E+00	1.337E-12
Na	4.558E-14	0.000E+00	1.970E-12
Ni	2.352E+02	1.924E+02	1.711E+02
S	7.340E-13	7.420E-18	8.592E-12
Si	1.314E+03	1.395E+03	1.469E+03
U	9.609E+03	9.608E+03	9.607E+03
Total (kg)	2.541E+04	3.308E+04	3.951E+04
Density (g/cm ³)	4.472	4.610	4.685

Table 32. Predicted Solution Elemental Composition (mole kg⁻¹) and pH in Selected Years for the Second Stage of Case 10 (n10b1011)

Years	103210	350290	634080
pH	5.66	5.80	6.16
Element			
Al	2.550E-06	1.393E-06	2.631E-07
B	1.239E-05	1.239E-05	1.239E-05
Ba	8.849E-07	7.121E-07	6.262E-07
Ca	8.249E-05	5.880E-06	3.179E-05
Cl	2.014E-04	2.014E-04	2.014E-04
Cr	1.564E-01	5.386E-02	7.056E-03
Cu	8.403E-04	3.212E-04	3.642E-05
F	1.148E-04	1.148E-04	1.148E-04
Fe	9.520E-12	6.521E-12	3.068E-12
C	4.100E-05	4.558E-05	5.954E-05
P	2.354E-08	2.674E-08	4.162E-08
K	1.008E-03	2.775E-04	6.977E-04
Li	6.915E-06	6.915E-06	6.915E-06
Mg	4.006E-05	3.080E-06	1.827E-05
Mn	1.169E-10	4.454E-11	4.970E-12
Mo	4.581E-03	3.731E-03	1.478E-06
N	3.305E-03	1.301E-03	2.795E-04
Na	1.992E-03	1.992E-03	1.992E-03
Ni	8.781E-02	3.344E-02	3.720E-03
S	6.061E-04	3.435E-04	2.096E-04
Si	6.802E-05	7.386E-05	6.097E-05
U	1.231E-05	8.877E-06	6.275E-06

Figure 24 shows the predicted partitioning of U during and shortly after degradation of the N Reactor Mark IA SNF in the second stage of Case 10. Complete degradation of the SNF is predicted to occur by the end of the first year of the second stage. The pH of the WP solution is suppressed slightly by SNF dissolution. Predicted soluble U concentration is kept at a fairly constant level by formation of schoepite ($\text{UO}_3 \cdot 2\text{H}_2\text{O}$) and $(\text{UO}_2)_3(\text{PO}_4)_2 \cdot 6\text{H}_2\text{O}$.

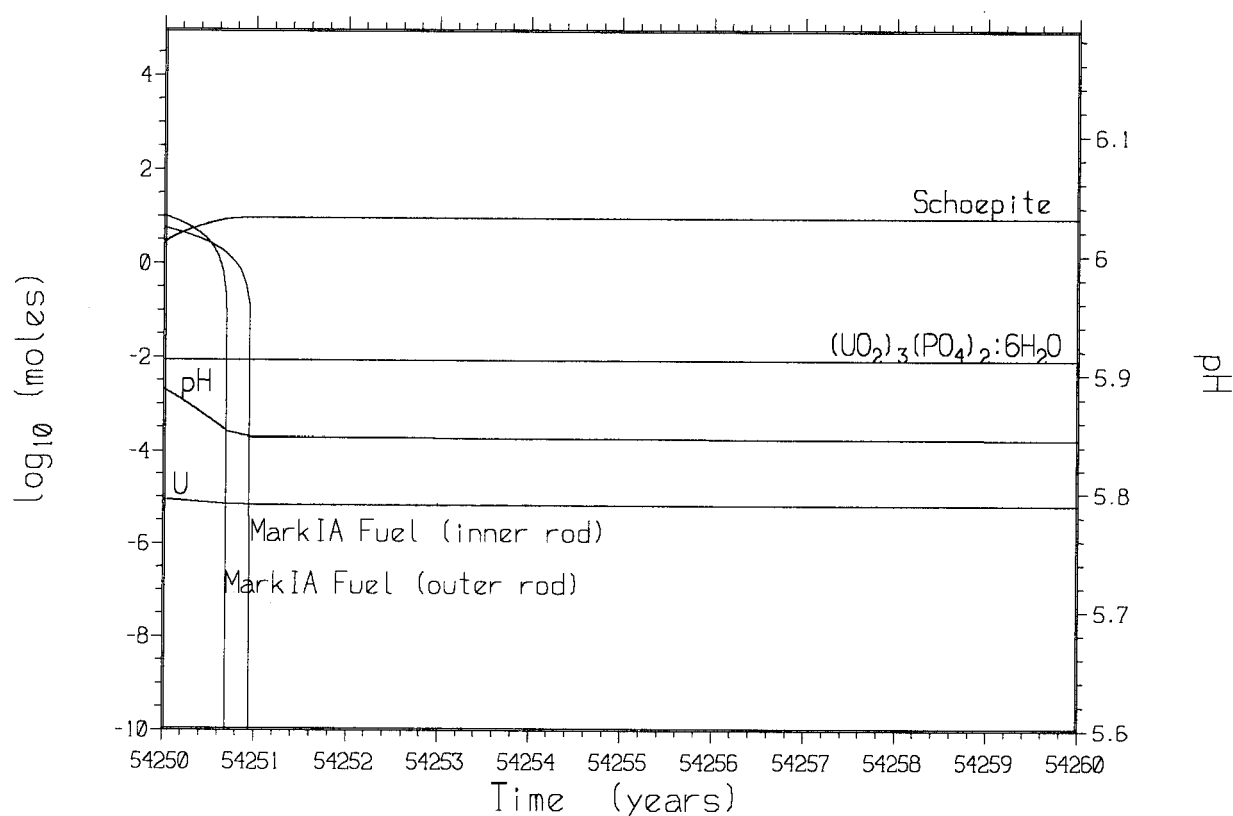


Figure 24. Predicted Concentration of N-Reactor SNF, Precipitated U Minerals, Total U in Solution, and pH as a Function of Time Early in the Second-Stage of Case 10 (n10b1011)

6. RESULTS

This document may be affected by technical product input information that requires confirmation. Any changes to the document that may occur as a result of completing the confirmation activities will be reflected in subsequent revisions. The status of the technical product input information quality may be confirmed by review of the DIRS database.

A principle objective of this calculation was to assess the chemical characteristics that might lead to the retention of U, Pu, and Np in a WP containing N Reactor (U-metal) SNF and HLW glass. Ten EQ6 reaction path calculations were carried out to assess the specific and coupled effects of SNF degradation, steel corrosion, HLW glass degradation, and fluid influx rate on U, Pu and Np mobilization. Corrosion product accumulation (primarily of iron oxide and smectite) and U, Pu, and Np mobilization were examined as well.

In these calculations the geochemical reaction takes place in the waste package solution maintained at a constant volume by steady inflow (drip rate) of fluid having the composition of J-13 well water and by removal of waste package solution at an equal rate. The composition of the waste package solution was determined by chemical reaction of the ions in solution, the elements released by the degradation of the initial waste package internal components, the minerals already precipitated, and by the mixing of the in-flowing fluid.

Case 10, the two-stage run, predicted that all of the U in the HLW glass (1.10% of the total U in the WP) would be lost from the WP for conditions of average steel, high HLW glass degradation rates and low drip rate simulated by the first stage of the run. Only 0.02% of the total U in the WP (from Mark IA SNF only) was predicted to be lost during the conditions simulated by the second stage of Case 10, with low steel, average SNF degradation rates, and low drip rate.

Cases 2 and 5 had predicted total U losses from the WP of close to 1%. Both these cases simulated conditions of high steel, average fuel, and low glass degradation rates, with a low drip rate.

Predicted loss of U from the WP was very low (0-0.16%) for the rest of the cases shown in Table 8.

Case 9 was run assuming a decayed composition for N Reactor Mark IA SNF (Table 2). Although predicted total loss of U was very low for this case (0.02%), predicted total loss of Np was high (~83%), and predicted total loss of Pu was low (<2%).

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8. ATTACHMENTS

Attachment I Documentation for Software Routine: volarea (32 pages)

Attachment II. Listing of Files on Compact Disk (5 pages)

Attachment III. One Compact Disk

KL 2/27/01

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SPECIAL INSTRUCTION SHEET

1. QA: QA

Page: 1 of: 1

Complete Only Applicable Items

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PC

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Windows

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CERTIFICATION**17. NAME (Print and Sign)**

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18. DATE:

1/31/01

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DC USE ONLY**23. DATE RECEIVED:**

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Teri M. McCoy

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ATTACHMENT I. DOCUMENTATION FOR SOFTWARE ROUTINE: VOLAREA

1. Listing of volarea code (pages I-2 through I-15)
2. Documentation of correct results (pages I-16 through I-18)
3. Sample input files for volarea: triga.in, test.in (pages I-19 through I-26)
4. Sample output files for volarea: triga.out, test.out (pages I-27 through I-30)
5. File atwts.in – necessary to run volarea (pages I-31 through I-32)

1. Listing of volarea (volarea.c on Attachment III)

/*volarea.c The purpose of this program is to calculate volumes and areas of shapes inside a waste package, and total those parameters for each material. This program operates stand alone to support the description of materials in the waste package; it also operates as the central part of the program eqsetup.c, which provides the input files for EQ6. In calculating the areas of prism and cylinder shells, the small area at the two ends, between the principal surfaces is not included. The program also calculates the moles of these shapes, according to composition and atomic weights. The moles and grams are also broken down by elements (which are called components, in keeping with EQ3/6 terminology). All the input is checked for consistency, except the number of components (elements) in each material. This program was initially developed before eqsetup.c, but the present version ultimately evolved as a reduction from eqsetup.c (for which the calculations for several of the more complex shapes were expanded and corrected). In eqsetup.c the separate printing of areas and volumes was for checking purposes only, hence the name of the file pointer in which they are printed, ferr.

The atomic weights are read in from the file atwts.in, using the subroutine bldatwts(). The elements use their international chemical symbols, with the first letter capitalized. The individual element weight values are for the natural isotopic mixture, except for Pu, which uses only the Pu-239 value, and

U, for which the principal isotopes of interest, (233, 235, 238, etc) can be entered individually (e.g. U-233, etc). These isotopic designations cannot be used in the present version of EQ3/6, so they are combined into a single element with symbol "U" having an average atomic weight calculated from the isotopic composition for that material. Two, or more, materials having different U isotope compositions will give slightly different molecular weights, but that doesn't matter because EQ3/6 uses its own value for atomic weight. The only parameters fed to EQ3/6 are moles remaining and molar volume, and this procedure ensures that they will be calculated in a consistent manner. For this system to work, all of the uranium isotopes must appear at the end of the material component list, but they may appear in any order. If the U isotopes are not confined to the end of the list, the program will quit with an appropriate error message.

The input, data.in, is read into an array of lines. This array is processed as

blocks of shapes, materials, and structures. The structures are built from shapes. Total mass and area are accumulated by tracing the shapes through their occurrences in the hierarchy of structures. Since each shape is associated with a single material (mtrl[i]) the mass and area of each material

can be accumulated along with accumulation for shapes. The principal results

of this process are the surface areas and moles for each material (the latter

being computed from the total mass for the material and its molecular weight),

since this is the primary input to EQ3/6. The shape components of a

structure have one line each. Each shape component line starts with the keyword component, followed by an equal sign and two fields. The first field is the name of the shape; the second field is the number of times the shape occurs in the structure. If this number is negative, the area and volume of the shape will be subtracted instead of added. This capability can be used to remove surfaces that are double counted when a solid structure is built of elementary shapes (such as a cone on top of a cylinder). Such a removal would be accomplished by subtracting a very thin plate having the surface area to be removed, but only an infinitesimal volume.

Areas for each material are accumulated in the routine bldmoles() for convenience. The area calculation incorporates a fracture factor, which can account for additional area due to internal fracturing, or for reduced exposed area due to protection such as intact cladding. Bookkeeping for these factor values is by shape. The designator in the file data.in is fracture_factor; for internal fracturing fracture_factor > 1; for simulating the protection by partly intact cladding fracture_factor < 1, and the value is inversely proportional to the amount of cladding protection left (which decreases with time).

There are two different forms for comments: (1) Anything after the obligatory fields in a keyword statement; (2) Any line that begins below with an '*'. The former are copied into the input array of lines, but not read because each keyword keys a read of only a limited number of fields. The latter are simply diverted into the debugging file, "junk.out".

The input file, "data.in" also has a capability like the FORTRAN namelist, provided that the variable name is preceded by a character less than ASCII 48 (decimal) and then a code line for reading that variable is inserted at an appropriate place in the program. This capability is presently implemented for the global variable literswater, for which a nominal value is assigned in the program, but which may be overridden by reading in the function bldmoles(). It is also implemented for the local

(to the subroutine bldmoles()) variable totalvolume, for which there is no default value because the parameter is not used at all if there is no namelist type input for it. For literswater the overriding value will be read if it occurs on any line of "data.in" provided the variable name (literswater) is prefixed by '\$'. If totalvolume is input, literswater will be calculated as the difference between the value of totalvolume (in liters) and the total volume of all the shapes (or all the materials) in the waste package.

The areas and volumes of the individual shapes are calculated in the subroutine buildvals(). The input parameters for each shape are straightforward, except for the spherical segment, which uses the parameters: sphereseg, altitude, largerad, smallerad. The use of the shape parameters is evident from the formulae for each shape in the switch statement of the subroutine bldvals().

```

*/

#include <stdio.h>
#include <float.h>
#include <string.h>
#include <stdlib.h>
#include <math.h>
#define PI (float)acos(-1)
#define MAXLOOP 100//max steps in trace of a shape through structure
hierarchy

//In the following array definitions i refers to the first index; j to the
second
FILE *fin,*ferr;
float findfloat(char*,int,int);//gets a float number from an array of strings
int findatwtindx(char*);//finds the index of an element in the atomic wt
table
float areas[100], //areas of individual shapes
    volumes[100],//volumes of individual shapes
    densities[100],//densities of individual materials
    degrades[100],//degradation rates of individual materials
    gramvals[30][30],//wt fraction of element j in material i
    compvals[30][30];//mole fraction of element j in material i
float mass[100]={0},//accumulates mass of material i
    mareas[20]={0},//accumulates area of material i
    literswater=3937, //water in bathtub, may be overridden in bldmoles().
    totalvolume,
    atwts[100],//array of atomic weights (read from atwts.in)
    nmoles[100]={0};//moles of material i
float bldmolwt(int),//builds molecular weight for each material
    mvolumes[20],//volume of a gram mole of material i
    fudgefac[20],//simply goes into EQ3/6 input; not now used for anything
    temperature=25, //selects thermo library for EQ3/6
    fracfac=1; //fracture factor for glass; should be an array for
additional mtrl
void etrim(char*),//trims blanks from end of a string
    ftrim(char*),//trims blanks from the beginning of a string
    errproc(char*,char*),//outputs two strings describing the error and
exits
    bldmoles(),//builds total masses and volumes for each material and
shape
    bldatwts();//builds the atomic weights array from atwts.in
char lines[800][100]; //array of input lines
char Uxnames[5][3]={"Ua","Ub","Uc","Ud","Ue"}; //dummy names for U
combinations
    //used internally; EQ3/6 reads only U for uranium.
float mwtUxvals[5]={0}; //Average atomic wt for all the U isotopes in this
mtrl
int numUwts=0; //Counts the number of materials with multiple U isotopes
char mtrl[100][20], //name of material for shape i
    compnames[30][30][20], //name (symbol) of element j of material i
    matnames[100][20], //name of material i
    shapenames[100][20]; //name of shape i
int buildvals(int,int), //builds array of lines from the input file, data.in
    numshapes=0, //number of shape blocks read from the lines array
    nummaterials=0, //number of material blocks read
    numstructures=0, //accumulates number of structures read from data.in

```

```

    getlof3(char*,char*,char*,int); //identify 3 basic input block types
(data.in)
int mindex=0, //tracks index for material block currently being read
    sindex=0, //tracks index for shape block currently being read
    findint(char*,int,int), //returns integer parameter when keyword is found
    findstr(char*,char*,int,int), //copies string parameter when keyword is
found
    stype[100], //shapetype for shape i
    ttype[100], //solid or shell for shape i
    occrrncs[100]={0}, //lowest level occurrences for shape i (from data.in)
    compnums[100], //number of elements in material i
    numlines, //number of lines read from data.in
    get2(char*,char*,char*), //splits a string at the third argument
    numelements, //number of elements read from atwts.in
    dislaw=3, //dissolution law index; simply passed to EQ3/6
    plaw=0, //precipitation law index; simply passed to EQ3/6
    loopcount; //counts steps in traceing a shape through the structure
hierarchy
char atnames[120][7]; //to be read from atwts.in, large to permit isotopes
char stnames[25][20], //array of structure names
    subnames[25][20][20]; //array of substructure names within structure
int nstocrrncs[25]={0}, //number of occurrences for this structure
    nsubocrrncs[25][20]={0}, //number of this substructure occurrences
    numsubs[25]; //number of substructures in this structure
int getcomps(int,char*); //returns num occurrences of this component in
structure
float atwts[100], //array of atomic weights
    afcone(float,float,float), //calculates area of the frustrum of a cone
    vfcone(float,float,float); //calculates volume of the frustrum of a cone
float matvolume[100]={0}, //accumulates total volume of material i
    smass[100]={0}, //accumulates total mass for all occurrences of shape i
    sarea[100]={0}, //accumulates total area for all occurrences of shape i
    svolume[100]={0}; //accumulates total volume for all occurrences of shape
i
int calcocrrncs(char*), //calc number of occurrences of a shape or structure
    foundc; //used as BOOL for occurrence of a shape

void main()
{int i=0, key;
char dummy[100];
for(i=0; i<20; i++) fudgefac[i]=1; //nominal value for unused parameter
bldatwts(); //read table of atomic weights
if((fin=fopen("data.in", "r"))==NULL)
    {printf("couldn't open input\n");
    exit(0);}
ferr=fopen("data.out", "w");
i=0; //the following line converts data.in to an internal array (lines[])
while((fgets(dummy, 100, fin)!=NULL) && (i<800))
    {if(dummy[0]=='#') fprintf(ferr, "%s", dummy); //divert comment to the output
    else strcpy(lines[i++], dummy);}
numlines=i;
i=0; //the following lines copy non-comment lines of data.in into lines[]
while((fgets(dummy, 90, fin)!=NULL) && (i<700))
    {if(dummy[0]=='#') fprintf(ferr, "%s", dummy); //divert comment to the output
    else strcpy(lines[i++], dummy);}
i=0; //The following loop builds a data array for each shape, material,
structure

```

```

while (i<=numlines)
    {while((lines[i][0]<48)&&(i<numlines))i++;//readthrough blank lines and any
        //namelist items

    if(i>=numlines)break;
    if((key=getlof3("shape","material","structure",i))==0)//type of block
        errproc("shape or material or structure",lines[i]);
    i+=buildvals(key,i);}//build array for this block
bldmoles();//comprehensive calculations of areas and volumes
printf("BY SHAPE\n");//diagnostic print to screen and file
fprintf(ferr,"BY SHAPE\n");
for(i=0;i<numshapes;i++)
    {printf("%s Area=%f Volume=%f Material=%s\n",
        shapenames[i],sarea[i],svolume[i],mtrl[i]);
    fprintf(ferr,"%s Area=%f Volume=%f Material=%s\n",
        shapenames[i],sarea[i],svolume[i],mtrl[i]);}
printf("\nBY MATERIAL\n");
fprintf(ferr,"\nBY MATERIAL\n");
for(i=0;i<nummaterials;i++)
    {printf("%s Density=%g Moles=%g Area=%g Volume=%g\n",
        matnames[i],densities[i],nmoles[i],mareas[i],matvolume[i]);
    fprintf(ferr,"%s Density=%g Moles=%g
        Area=%g Volume=%g\n",
        matnames[i],densities[i],nmoles[i],mareas[i],matvolume[i]);}
printf("%d input lines read\n",numlines);}

```

```

//Determines if line i starts a block of one of the 3 types
int getlof3(char *s1,char *s2,char *s3,int i)
{int j;
char *p,stag[20];
if((p=strchr(lines[i],'))==NULL) return 0;
j=(int)(p-lines[i]);
strncpy(stag,lines[i],j);
stag[j]='\0';
etrim(stag);
if(strcmp(s1,stag)==0) return 1;
else if (strcmp(s2,stag)==0) return 2;
else if (strcmp(s3,stag)==0) return 3;
else return 0;}

```

```

//Builds data array for each block of data.in, including calculation of
//volume and surface area for each shape.
int buildvals(int type,int row)
{int i=1,j,numgrp,k;//number of lines in this block/group
char tempstr[100],stag[20];//to hold substrings for further processing
float x,y,z,r,t,t1,t2,f,a,turns,dr,rl,r2;//for shape dimensions
while((getlof3("shape","material","structure",row+i)==0)&&(lines[row+i][0]>=48)&&
    (row+i<=numlines))i++; //find number of lines in this block
numgrp=i;//number of lines in this group/block
strcpy(tempstr,lines[row]);
if(type==2) //build array for this material block
    {get2(stag,tempstr,'=');
    strcpy(matnames[mindex],tempstr);
    if((densities[mindex]=findfloat("density",row,numgrp))==0)

```

```

        errproc("no density for ",tempstr);
    if((j=findstr(stag,"degrate",row,numgrp))==0)
        errproc("no degradation rate for ",tempstr);
    strcpy(tempstr,lines[row+j]);
    i=get2(stag,tempstr,'=');
    strcpy(tempstr,lines[row+j]+i+1);
    if((k=get2(stag,tempstr,' '))===-1)
        errproc("missing degrate units for ",matnames[mindex]);
    degrades[mindex]=(float)atof(stag);
    if(strcmp(tempstr,"moles")==0)degrades[mindex]*= -1;//flag for alternate
units
    else if (strcmp(tempstr,"grams")!=0) //used in bldmoles
        errproc("invalid name for degradation rate units for ",matnames[mindex]);
    if((compnums[mindex]=findint("numelements",row,numgrp))==0)
        errproc("no component number for ",tempstr);
    if(compnums[mindex]+3>numgrp)
        {printf("compnums=%d numgrp=%d nummtrls=%d\n",
                compnums[mindex],numgrp,nummaterials);
        errproc("too many lines",tempstr);}
    for(j=0;j<compnums[mindex];j++) //elements must be last
        {strcpy(tempstr,lines[row+4+j]); //four lines for the previous items
        get2(stag,tempstr,'=');
        strcpy(compnames[mindex][j],stag);
        gramvals[mindex][j]=(float)atof(tempstr)/100;
        compvals[mindex][j]=
            gramvals[mindex][j]/atwts[findatwtindx(compnames[mindex][j])];}
    mindex++; //processing for this material completed
    nummaterials++;}
else if(type==1)//processing for a shape block
    {get2(stag,tempstr,'=');
    strcpy(shapenames[sindex],tempstr);
    if((j=findstr(stag,"type",row,numgrp))==0) errproc("no type for
shape",tempstr);
    strcpy(tempstr,lines[row+j]);
    i=get2(stag,tempstr,'=');
    strcpy(tempstr,lines[row+j]+i+1);
    get2(stag,tempstr,' ');
    if(strcmp(stag,"cube")==0) stype[sindex]=0;
    else if(strcmp(stag,"box")==0) stype[sindex]=1;
    else if(strcmp(stag,"cylinder")==0) stype[sindex]=2;
    else if(strcmp(stag,"sphere")==0) stype[sindex]=3;
    else if(strcmp(stag,"rtprism")==0) stype[sindex]=4;
    else if(strcmp(stag,"etprism")==0) stype[sindex]=5;
    else if(strcmp(stag,"hprism")==0) stype[sindex]=6;
    else if(strcmp(stag,"helix")==0) stype[sindex]=7;
    else if(strcmp(stag,"cone frustrum")==0) stype[sindex]=8;
    else if(strcmp(stag,"sphere seg")==0) stype[sindex]=9;
    else if(strcmp(stag,"pie")==0) stype[sindex]=10;
    else errproc("No shape type ",stag);
    if(strcmp(tempstr,"shell")==0) ttype[sindex]=0;
    else ttype[sindex]=1;
    occrrncs[sindex]=findint("occurrences",row,numgrp);//returns zero if the
//shape occurs in a structure
    if(findstr(stag,"matname",row,numgrp)==0) //material for this shape
        errproc("no material for shape",shapenames[sindex]);
    strcpy(mtrl[sindex],stag);
    if((x=findfloat("fracture_factor",row,numgrp))>0) fracfac=x;

```

```

else fracfac=1;
switch(stype[sindex])//get the appropriate dimensions for this shape
    //and calculate its area and volume
{case 0:                                     //cube
    if((x=findfloat("side",row,numgrp))==0)errproc("no side for ",stag);
    if(ttype[sindex]==1) //for solid
        {areas[sindex]=6*(float)pow(x,2);
        areas[sindex]*=fracfac;
        volumes[sindex]=(float)pow(x,3);}
    else //for shell
        {if((t=findfloat("thickness",row,numgrp))==0)
            errproc("no thickness for cube shell","");
        areas[sindex]=6*(float)((pow(x,2)+pow(x-2*t,2)));
        areas[sindex]*=fracfac;
        volumes[sindex]=(float)((pow(x,3)-pow(x-2*t,3)));}
    break;
case 1:                                     //rectangular box
    if((x=findfloat("side1",row,numgrp))==0)errproc("no side1 for ",stag);
    if((y=findfloat("side2",row,numgrp))==0)errproc("no side2 for ",stag);
    if((z=findfloat("side3",row,numgrp))==0)errproc("no side3 for ",stag);
    if(ttype[sindex]==1)
        {areas[sindex]=2*(x*y+y*z+x*z);
        areas[sindex]*=fracfac;
        volumes[sindex]=x*y*z;}
    else
        {if((t=findfloat("thickness",row,numgrp))==0)
            errproc("no thickness for box shell","");
        areas[sindex]=2*(x*y+y*z+x*z)
        +2*(float)((x-2*t)*(y-2*t)+(y-2*t)*(z-2*t)+(z-2*t)*(x-2*t));
        areas[sindex]*=fracfac;
        volumes[sindex]=x*y*z-(x-2*t)*(y-2*t)*(z-2*t);}
    break;
case 2:                                     //cylinder
    if((r=findfloat("radius",row,numgrp))==0)errproc("no radius for
",stag);
    if((z=findfloat("length",row,numgrp))==0)errproc("no length for
",stag);
    if(ttype[sindex]==1)
        {areas[sindex]=2*PI*r*z+2*PI*r*r;
        areas[sindex]*=fracfac;
        volumes[sindex]=PI*r*r*z;}
    else
        {if((t=findfloat("thickness",row,numgrp))==0)
            errproc("no thickness for cylinder shell","");
        areas[sindex]=2*PI*r*z+2*PI*(r-t)*z;
        areas[sindex]*=fracfac;
        volumes[sindex]=PI*r*r*z-PI*(r-t)*(r-t)*z;}
    break;
case 3:                                     //sphere
    if((r=findfloat("radius",row,numgrp))==0)errproc("no radius for
",stag);
    if(ttype[sindex]==1)
        {areas[sindex]= 4*PI*r*r;
        areas[sindex]*=fracfac;
        volumes[sindex]=4*PI*r*r*r/3;}
    else
        {if((t=findfloat("thickness",row,numgrp))==0)

```



```

        errproc("no thickness for spherical shell","");
        areas[sindex]=4*PI*(r*r+(r-t)*(r-t));
        areas[sindex]*=fracfac;
        volumes[sindex]=4*PI*r*r*r/3-4*PI*(r-t)*(r-t)*(r-t)/3;}
    break;
case 4:                                     //isocetes right triangle prism
    if((x=findfloat("side",row,numgrp))==0)errproc("no side for ",stag);
    if((z=findfloat("length",row,numgrp))==0)errproc("no length for
",stag);
    f=2+(float)sqrt(2);                    // also 1+cot(22.5deg)
    if(ttype[sindex]==1)
        {areas[sindex]=x*f*z+x*x; //area of sides and ends
        areas[sindex]*=fracfac;
        volumes[sindex]=x*x*z/2;}
    else
        {if((t=findfloat("thickness",row,numgrp))==0)
            errproc("no thickness for right triangle prism shell","");
        areas[sindex]=x*f*z+(x-f*t)*f*z;
        areas[sindex]*=fracfac;
        volumes[sindex]=(x*x-(x-f*t)*(x-f*t))*z/2;}
    break;
case 5:                                     //equilateral triangle prism
    if((x=findfloat("side",row,numgrp))==0)errproc("no side for ",stag);
    if((z=findfloat("length",row,numgrp))==0)errproc("no length for
",stag);
    if(ttype[sindex]==1)
        {areas[sindex]=3*x*z+x*x*(float)sqrt(3)/2; //area of sides and ends
        areas[sindex]*=fracfac;
        volumes[sindex]=x*x*z*(float)sqrt(3)/4;}
    else
        {if((t=findfloat("thickness",row,numgrp))==0)
            errproc("no thickness for equilateral triangle prism shell","");
        f=2*(float)sqrt(3);                // also 2cot(30deg)
        areas[sindex]=3*x*z+3*(x-f*t)*z;
        areas[sindex]*=fracfac;
        volumes[sindex]=x*x*z*(float)sqrt(3)/4-(x-f*t)*(x-
f*t)*z*(float)sqrt(3)/4;}
    break;
case 6:                                     //hexagon based prism
    if((x=findfloat("side",row,numgrp))==0)errproc("no side for ",stag);
    if((z=findfloat("length",row,numgrp))==0)errproc("no length for
",stag);
    if(ttype[sindex]==1)
        {areas[sindex]=6*x*z+3*x*x*(float)sqrt(3); //area of sides and ends
        areas[sindex]*=fracfac;
        volumes[sindex]=6*x*x*z*(float)sqrt(3)/4;}
    else
        {if((t=findfloat("thickness",row,numgrp))==0)
            errproc("no thickness for hexagonal prism shell","");
        f=2/(float)sqrt(3);                // also 2tan(30deg)
        areas[sindex]=6*x*z+6*(x-f*t)*z;
        areas[sindex]*=fracfac;
        volumes[sindex]=6*x*x*z*(float)sqrt(3)/4
        -6*(x-f*t)*(x-f*t)*z*(float)sqrt(3)/4;}
    break;
case 7:                                     //helix
    if((turns=findfloat("turns",row,numgrp))==0)

```

```

        errproc("no turns for ",stag);
    if((a=findfloat("smallradius",row,numgrp))==0)
        errproc("no smallradius for ",stag);
    if((r=findfloat("largeradius",row,numgrp))==0)
        errproc("no largeradius for ",stag);
    if((z=findfloat("altitude",row,numgrp))==0)errproc("no altitude for
",stag);
    if(ttype[sindex]==1)
        {areas[sindex]=2*PI*a*(float)sqrt(z*z+pow(2*PI*r*turns,2));
        volumes[sindex]=areas[sindex]*a/2;
        areas[sindex]*=fracfac;} //now adjust area for fractures
    else errproc("no helical shell","");
    break;
/* Alternative area=PI*a*z/sin(alph), where alph is the dihedral angle
between
the horizontal and any tangent plane of the helix. This formulation would
require only 3 input parameters.*/

case 8: //frustum of cone
    if((z=findfloat("altitude",row,numgrp))==0)errproc("no altitude for
",stag);
    if((r1=findfloat("bottomrad",row,numgrp))==0)
        errproc("no bottom radius for ",stag);
    if((r2=findfloat("toprad",row,numgrp))==0)
        errproc("no top radius for ",stag);
    if(ttype[sindex]==1)
        {areas[sindex]=afcone(r1,r2,z);
        areas[sindex]*=fracfac;
        volumes[sindex]=vfcone(r1,r2,z);}
    else
        {if((t=findfloat("thickness",row,numgrp))==0)
            errproc("no thickness for cone frustum shell","");
        dr=t*(float)sqrt(1+pow(r1-r2,2)/z/z);
        areas[sindex]=afcone(r1,r2,z)+afcone(r1-dr,r2-dr,z);
        areas[sindex]-=PI*(r2*r2+r1*r1);//remove top and bottom from outer
        areas[sindex]-=PI*(float)(pow(r2-dr,2)+pow(r1-dr,2));//and from inner
        areas[sindex]*=fracfac;
        volumes[sindex]=vfcone(r1,r2,z)-vfcone(r1-dr,r2-dr,z);}
    break;
case 9://segment of a sphere; large and small radii of the segment (r1,r2)
    //are measured from an axis of the sphere; the radius of the
    //actual sphere is derived from r1,r2, and the altitude, z. For a
    //shell, the thickness, t, is for the material itself. The
    //projections of this thickness onto the large and small segment
    //planes are derived as the variables t1,t2.
    if((z=findfloat("altitude",row,numgrp))==0)//distance between top *
bottom
        errproc("no altitude for ",stag);
    if((r1=findfloat("largerad",row,numgrp))==0)
        errproc("no largerad for ",stag);
    if((r2=findfloat("smallrad",row,numgrp))==0)
        errproc("no smallrad for ",stag);
    r=(float)sqrt(r2*r2+pow(r1*r1-r2*r2+z*z,2)/(4*z*z));//radius of sphere
    if(ttype[sindex]==1)
        {areas[sindex]=PI*(2*r*z+r1*r1+r2*r2); //area of upper and lower too
        areas[sindex]*=fracfac;
        volumes[sindex]=PI*z*(z*z+3*(r1*r1+r2*r2))/6;}

```

```

else
    {if((t=findfloat("thickness",row,numgrp))==0)
        errproc("no thickness for sphere segment shell","");
        t1=t*r/r1;//project thickness onto large plane of segment
        t2=t*r/r2;//onto the small plane of segment
        areas[sindex]=2*PI*(r*z+(r-t)*(z-t));
        areas[sindex]*=fracfac;
        volumes[sindex]=PI*z*(z*z+3*(r1*r1+r2*r2))/6
            -PI*(z-t)*((z-t)*(z-t)+3*((r1-t1)*(r1-t1)+
                (r2>t2?(r2-t2)*(r2-t2):0)))/6;}//If the small segment
radius                                     //is less than the projected thickness, it is omitted
from                                     //the volume calculation. This is the usual
condition,                             //since the segment is usually a cap.

        break;
        case 10://pie shape;radius, length, angle, calc for solid only
            if((z=findfloat("length",row,numgrp))==0)//distance between top *
bottom
                errproc("no length for ",stag);
                if((r=findfloat("radius",row,numgrp))==0)
                    errproc("no radius for ",stag);
                if((a=findfloat("angle",row,numgrp))==0)
                    errproc("no angle for ",stag);
                areas[sindex]=(2*PI*r*r+2*PI*r*z)*(a/360)+2*r*z;
                volumes[sindex]=PI*r*r*z*a/360;
                sindex++; //processing for this shape complete
                numshapes++;}
else if (type==3)//build array for this structure
    {get2(stag,tempstr,'=');
    strcpy(stnames[numstructures],tempstr);
    for(i=1;i<numgrp;i++)
        if((j=getcomps(row+i,stag))!=0)
            {nsubocrrncs[numstructures][numsubs[numstructures]]=j;
            strcpy(subnames[numstructures][numsubs[numstructures]],stag);
            numsubs[numstructures]++;}
    nstocrrncs[numstructures]=findint("occurrences",row,numgrp);
    numstructures++;}
return numgrp;}

//for a component line of a structure block, returns occurrences
int getcomps(int rw,char* retstr)//duplicates findstr(), but for a single
line
{int i;
char *p,stag[20],tempstr[100];
if(strncmp(lines[rw],"component",9)!=0) return 0;
strcpy(tempstr,lines[rw]);
p=strchr(lines[rw],'=');
i=(int)(p-lines[rw]);
strcpy(tempstr,lines[rw]+i+1);
get2(stag,tempstr,' ');
strcpy(retstr,stag);
return atoi(tempstr);}

float afcone(float rl,float ru,float h) //area of the frustrum of a cone
{float area;

```

```

area=PI*(rl*rl+ru*ru); //top and bottom disks
area+= PI*h*(rl+ru)*(float)sqrt(1+rl*rl*pow(1-ru/rl,2)/h/h);
return area;
}

```

```

float vfcone(float rl,float ru,float h) //volume of the frustrum of a cone
{return PI*h*(rl*rl+rl*ru+ru*ru)/3;}

```

```

//Find the float numerical value in a statement that begins with the
specified
//keyword (keyst).
float findfloat(char* keyst,int startrow,int numRows)
{int i=0;
char stag[20],tempstr[100];
while ((strncmp(keyst,lines[startrow+i],strlen(keyst))!=0)&&(i<=numRows))
    i++;
if (i==numRows+1) return 0;
strcpy(tempstr,lines[startrow+i]);
get2(stag,tempstr,'=');
return (float)atof(tempstr);}

```

```

//Find the int numerical value in a statement that begins with the specified
//keyword (keyst).
int findint(char* keyst,int startrow,int numRows)
{int i=0;
char stag[20],tempstr[100];
while
((strncmp(keyst,lines[startrow+i],strlen(keyst))!=0)&&(i<=numRows))i++;
if (i>=numRows) return 0;
strcpy(tempstr,lines[startrow+i]);
get2(stag,tempstr,'=');
return atoi(tempstr);}

```

```

//This function searches for a key at the start of the line and copies the
//string (containing no blanks) following the equal sign into retstr.
//It returns zero if the keysting is not found, otherwise the row in which
found.

```

```

//It is analogous to findfloat and findint, but does not return the
//information, since C cannot assign to a character string.
int findstr(char *retstr,char *keyst,int startrow,int numRows)
{int i=0;
char stag[20],tempstr[100];
while
((strncmp(keyst,lines[startrow+i],strlen(keyst))!=0)&&(i<=numRows))i++;
if (i==numRows) return 0;
strcpy(tempstr,lines[startrow+i]);
get2(stag,tempstr,'=');
strcpy(retstr,tempstr);
return i;}

```

```

//copies the strings occurring before and after the separator c ('=' or ' ')
int get2(char *stag,char *sdummy,char c)
{int i,index,len;
char *p;
len=strlen(sdummy);
p=strchr(sdummy,c);
i=(int) (p-sdummy);

```

```

index=i;
strncpy(stag,sdummy,i);
stag[i]='\0';
etrim(stag);
strcpy(sdummy,p+1);
i=0;
while((sdummy[i]==' ') && (i<len)) i++;
if((i==len) || (sdummy[i]=='\n')) return -1; //can't find a second word on rt
side
strcpy(sdummy,sdummy+i);
while((sdummy[i]!=' ') && (sdummy[i]!='\0') && (sdummy[i]!='\n')) i++;
sdummy[i]='\0';
ftrim(sdummy);
return index;}

void etrim(char *dummy) //trims trailing blanks
{int i;
i=strlen(dummy)-1;
while(dummy[i]<=32) i--;
dummy[i+1]='\0';}

//trims leading blanks
void ftrim(char *dummy) //use in get2 so that fdummy starts with a non-
blank
{int i=0;
while(dummy[i]<=32) i++;
strcpy(dummy,dummy+i);}

//prints a two string error message
void errproc(char *str1,char *str2)
{printf("%s %s\n",str1,str2);
exit(0);}

//accumulate total mass, area, and volume for shapes[j] and mtrls[i]
void bldmoles()
{int i,j,k,normflag=0;
float x,molwt,checkvol=0,totalvolume,compsum[50]={0};
if((x=findfloat("$literswater",0,numlines))>0) literswater=x;
else if((x=findfloat("$totalvolume",0,numlines))>0)
{totalvolume=x;
normflag=1;}
for(i=0;i<nummaterials;i++)
{checkvol=0;
for(j=0;j<numshapes;j++)
if(strcmp(matnames[i],mtrl[j])==0)
{if((k=occcrrncs[j])==0)
{foundc=0;
loopcount=0;
k=calcocccrrncs(shapenames[j]);
if((foundc==0) || (k==0)) errproc("no occurrences for
",shapenames[j]);}
mass[i]+=volumes[j]*k*densities[i];
mareas[i]+=areas[j]*k;
checkvol+=volumes[j]*k;
smass[j]+=volumes[j]*k*densities[i];
sarea[j]+=areas[j]*k;
svolume[j]+=volumes[j]*k;}
}
}

```

```

if(normflag==1)
    {literswater=totalvolume-checkvol;
    fprintf(ferr,"Total solids volume=%.4e Void volume=%.4e\n",
        checkvol,literswater);}
    fprintf(ferr,"Composition of %s in grams and moles, check vol=%f\n",
        matnames[i],checkvol);
    fprintf(ferr,"%15s%15s%15s\n","Name","Grams","Moles");
    for(j=0;j<compnums[i];j++)
        compsum[i]+=mass[i]*gramvals[i][j];
    for(j=0;j<compnums[i];j++)
        fprintf(ferr,"%15s%15.3e%15.3e\n",compnames[i][j],
            mass[i]*gramvals[i][j]*100/compsum[i],
            mass[i]*compvals[i][j]*100/compsum[i]);
    molwt=bldmolwt(i);
    degrades[i]/=100;
    if (degrades[i]<0) degrades[i]*= -molwt;//only use for molwt now
    mareas[i]/=literswater;
    mvolumes[i]=100/densities[i]; //used to be molwt/densities[i];
    matvolume[i]=mass[i]/densities[i];
    nmoles[i]=mass[i]/100/literswater;}} //used to be mass[i]/molwt/literswater

int calcocrrncs(char* sstr)//num occurrences of sstr in this level or above
//additive counter chains beginning at this level; multiplicative counter
//for propagating a chain up the structure hierarchy
{int i,j,k,no;//multiplicative counter
    account=0;//additive counter
    kk;
    for(i=0;i<numstructures;i++)
        {j=0;
        while((j<numsubs[i])&&(kk=strcmp(subnames[i][j],sstr))!=0)j++;
        if(kk==0)
            {no=nsubocrrncs[i][j];
            if((k=nstocrrncs[i])>0)
                {no*=k;
                foundc=1;} //found the end of the structure chain for this shape
            else
                {no*=calcocrrncs(stnames[i]);
                loopcount++;
                if(loopcount>MAXLOOP) errproc("Idiot, you have an occurrence
loop", "");}
            account+=no;}}
    return account;}

//builds the molecular weight (from the atwts table) for each material block
//of the input; if the material contains more than one isotope of U, these
//are averaged together, and the individual isotope entries in the material
//components are collapsed into one atomic weight and one symbol using the
//variable Uxnames={"Ua",...}
float bldmolwt(int ndx)
{int j,k,kk,numUs,hasIso=0;
char *p;
float mwt=0;
j=0;
while ((j<compnums[ndx])&&((p=strchr(compnames[ndx][j],'-'))==NULL))j++;
if(p!=NULL)
    {hasIso=1;
    while ((j<compnums[ndx])&&((p=strchr(compnames[ndx][j],'-'))!=NULL))j++;

```

```
    if(j!=compnums[ndx]) errproc("", "all isotopes must come last");}
if(hasIso==1)
{
    j=compnums[ndx]-1;
    while((p=strchr(compnames[ndx][j], '-')!=NULL)
        {if((p-compnames[ndx][j])!=1) || (compnames[ndx][j][0]!='U'))
            errproc(compnames[ndx][j], "is a strange isotope");
            j--;}
    if(j<compnums[ndx]-1)
    {
        numUs=compnums[ndx]-j-1;
        for (k=0; k<numUs; k++)
        {
            kk=compnums[ndx]-k-1;
            mwtUxvals[numUwts]+=
                atwts[findatwtindx(compnames[ndx][kk])]*gramvals[ndx][kk];}
        compnums[ndx]-=numUs-1;
        strcpy(compnames[ndx][compnums[ndx]-1], Uxnames[numUwts]);
        atwts[numelements]=mwtUxvals[numUwts];
        strcpy(atnames[numelements], Uxnames[numUwts]);
        numelements++;
        numUwts++;}
    for(j=0; j<compnums[ndx]; j++)//cycle through the elements of this material
        mwt+=atwts[findatwtindx(compnames[ndx][j])]*gramvals[ndx][j];
    return mwt;}

void bldatwts()//builds the table of atomic weights from the file atwts.in
{
    int i=0;
    if((fin=fopen("atwts.in", "r"))==NULL)
        {printf("couldn't open atomic wts input\n");
            exit(0);}
    while(fscanf(fin, "%s %f", atnames[i], &atwts[i])!=EOF) i++;
    numelements=i;
    fclose(fin);}

int findatwtindx(char elname[7]) //finds the atomic wt for the named element
{
    int i=0;
    while((i<numelements)&&(strcmp(elname, atnames[i])!=0)) i++;
    if (i==numelements)
        errproc(elname, " is not in the table of atomic weights");
    return i;}

```

2. Documentation of correct results

Table I-1 presents the volume and area of various shapes calculated by volarea using input files "triga.in" and "test in" (Attachment III), with the results presented in output files "triga.out" and "test.out" (Attachment III). The hand calculations differed slightly from the volarea results for the volume of the spherical segment shell but only by less than 1%. The hand calculations confirmed that the volarea results are correct.

Volarea was documented previously, as well as here, using the data from the TRIGA geochemistry calculation, Ref. 21 (MOL.19991118.0050). This documentation is simply to show that the volarea routine works for its intended purpose, which is to calculate the areas and volumes of different waste package components.

Table I-1. Comparison of Volarea and Hand Calculation Results to Confirm Volarea Software Routine

Item Name	Shape with Number of Items in Parentheses	Volume				Area			
		Volarea Result		Hand Calculated		Volarea Result		Calculated	
		Volume All Items	Volume 1 Item	Volume Equation ^a	Volume	Area All Items	Area 1 Item	Area Equation ^a	Area
sstubes	cylinder shell (111)	88401	796	$\text{volume} = \pi * \text{rout}^2 * L - \pi * (\text{rout} - \text{thickness})^2 * L$	796	319297	2877	$\text{area} = 2 * \pi * \text{rout} * L + 2 * \pi * (\text{rout} - \text{thickness}) * L$	2877
baseplate	cyldinder solid (3)	4062	1354	$\text{volume} = \pi * r^2 * L$	1354	8933	2978	$\text{area} = 2 * \pi * r * L + 2 * \pi * r^2$	2978
bracket	box solid (36)	3970	110	$\text{volume} = L1 * L2 * L3$	110	11400	317	$\text{area} = 2 * L1 * L2 + 2 * L1 * L3 + 2 * L2 * L3$	317
fitting	cone frustum solid (222)	7453	34	$\text{volume} = 1/3 * (\text{areabase} + \text{areatop} + (\text{areabase} * \text{areatop})^{1/2}) * \text{alt}$	34	14619	66	$\text{area} = \pi * (\text{rbase}^2 + \text{rtop}^2 + \text{rtop}^2 + 1/2 * (2 * \pi * (\text{rbase} + 2 * \pi * \text{rtop}) * (\text{alt}^2 + (\text{rbase} - \text{rtop})^2)^{1/2}))$	66
impactplates 2	sphereseg solid (2)	8426	4213	$\text{volume} = 1/6 * \pi * \text{alt}^3 * (\text{rtop}^2 + 3 * \text{rbot}^2 + \text{alt}^2)$	4213	6382	3191	$R (\text{radius of circle}) = \sqrt{1 / (4 * \text{alt}^2) * (\text{rbot}^2 - \text{rtop}^2 + \text{alt}^2)^2 + \text{rtop}^2}$ $\text{area} = 2 * \pi * R * \text{alt} + \pi * (\text{rbot}^2 + \text{rtop}^2)$	45 3191
dishheadsph	sphereseg shell (2) (like a cap)	2995	1498	$\text{Rout} (\text{radius of outer circle}) = \sqrt{1 / (4 * \text{alt}^2) * (\text{rbot}^2 - \text{rtop}^2 + \text{alt}^2)^2 + \text{rtop}^2}$ $\text{volume} = 1/3 * \pi * \text{alt}^2 * (3 * \text{Rout} - \text{alt}) - 1/3 * \pi * (\text{alt} - t)^2 * (3 * (\text{Rout} - t) - (\text{alt} - t)) - 1/2 * b * h^2 * \pi * (r - t)$	47 1487	6389	3194	$\text{area} = 2 * \pi * \text{Rout} * \text{alt} + 2 * \pi * (\text{Rout} - t) * (\text{alt} - t)$	3194
shape1	cube solid (10)	80	8	$\text{volume} = \text{side}^3$	8	240	24	$\text{area} = 6 * \text{side}^2$	24
shape2	cube shell (10)	22	2	$\text{volume} = \text{side}^3 - (\text{side} - 2 * t)^3$	2	434	43	$\text{area} = 6 * \text{side}^2 + 6 * (\text{side} - 2 * t)^2$	43
shape3	box shell (10)	48	5	$\text{volume} = L1 * L2 * L3 - (L1 - 2 * t) * (L2 - 2 * t) * (L3 - 2 * t)$	5	970	97	$\text{area} = 2 * (L1 * L2 + L1 * L3 + L2 * L3 + (L1 - 2 * t) * (L2 - 2 * t) + (L1 - 2 * t) * (L3 - 2 * t) + (L2 - 2 * t) * (L3 - 2 * t))$	97
shape4	sphere solid (10)	335	34	$\text{volume} = 4/3 * \pi * r^3$	34	503	50	$\text{area} = 4 * \pi * r^2$	50
shape5	sphere shell (10)	48	5	$\text{volume} = 4/3 * \pi * (r^3 - (r - t)^3)$	5	956	96	$\text{area} = 4 * \pi * (r^2 + (r - t)^2)$	96
shape6	rtprism solid (10)	400	40	$\text{volume} = 1/2 * \text{side}^2 * \text{length}$	40	1406	141	$\text{area} = 2 * (1/2 * \text{side}^2) + (\text{side} + \text{side} + \text{hypotenuse}) * \text{length}$	141

Table I-1. Comparison of Volarea and Hand Calculation Results to Confirm Volarea Software Routine (Continued)

Item Name	Shape with Number of Items in Parentheses	Volume				Area			
		Volarea Result		Hand Calculated		Volarea Result		Calculated	
		Volume All Items	Volume 1 Item	Volume Equation ^a	Volume	Area All Items	Area 1 Item	Area Equation ^a	Area
shape7	rtprism shell (10)	125	12	$\text{volume} = 1/2 * \text{length} * (\text{side}^2 - (\text{side} - t / \tan(22.5 * \text{PI}() / 180))^2)$	12	2498	250	$\text{hypotenuseinside} = \sqrt{2 * (\text{side} - t / \tan(22.5 * \text{PI}() / 180))^2}$ $\text{area} = (\text{side} + \text{side} + \text{hypotenuse}) * \text{length} + (\text{side} - t / \tan(22.5) + \text{side} - t / \tan(22.5) + \text{hypotenuseinside}) * \text{length}$	250
shape8	etprism solid (10)	346	35	$\text{volume} = 1/4 * \text{side}^2 * (3^{1/2}) * \text{length}$	35	1234	123	$\text{area} = 3 * \text{side} * \text{length} + 2 * 1/4 * \text{side}^2 * (3^{1/2})$	123
shape9	etprism shell (10)	110	11	$\text{volume} = 3^{1/2} / 4 * \text{length} * (\text{side}^2 - (\text{side} - 2 * t * 3^{1/2})^2)$	11	2192	219	$\text{area} = 3 * \text{length} * (\text{side} + \text{side} - 2 * t * 3^{1/2})$	219
shape10	hprism solid (10)	2079	208	$\text{volume} = 3/2 * \text{side}^2 * (3^{1/2}) * \text{length}$	208	2607	261	$\text{area} = 3 * \text{side}^2 * (3^{1/2}) + 6 * \text{side} * \text{length}$	261
shape11	hprism shell (10)	233	23	$\text{volume} = 3/2 * 3^{1/2} * \text{length} * (\text{side}^2 - (\text{side} - 2 * t * 3^{1/2})^2)$	23	4661	466	$\text{area} = 6 * \text{length} * (\text{side} + \text{side} - 2 * t * (3^{1/2}))$	466
shape12	helix solid (10)	124	12	$\text{volume} = \# \text{turns} * 2 * \text{PI}^2 * \text{larger} * \text{smallr}^2$	12	2472	247	$\text{area} = \# \text{turns} * 4 * \text{pi}^2 * \text{larger} * \text{smallr}$	247
shape13	cone frustum shell (10)	154	15	$a \text{ (horizontal thickness)} = t / \sin(\text{inv tan}(h / ((\text{rbot} - \text{rtop})))$ $\text{volume} = 1/3 * \text{PI} * H * ((\text{rbot}^2 + \text{rtop}^2 + \text{rbot} * \text{rtop}) - ((\text{rbot} - a)^2 + (\text{rtop} - a)^2 + (\text{rtop} - a) * (\text{rbot} - a))$	0.20 15	1537	154	$s \text{ (slant height)} = (a^2 + (\text{rbase} - \text{rtop})^2)^{1/2}$ $\text{area} = 1/2 * s * (2 * \text{PI}() * \text{rbase} + 2 * \text{PI}() * \text{rtop} + 2 * \text{PI}() * (\text{rbase} - a) + 2 * \text{PI}() * (\text{rtop} - a))$	5 154

Source: ^aReferences 64 and 16

3. Sample input file: triga.in

```
structure=fuelrods
component=trigaIA 111
occurrences=1
```

```
structure=trigaIA
component=fuel 1
component=centerrod 1
component=upperreflector 1
component=lowerreflector 1
component=cladding 1
component=fitting 2
```

```
$literswater=4100
```

```
shape=fuel
type=cylinder shell
length=38.1
radius=1.82245
thickness=1.50495
matname=UZrHFLIP
```

```
shape=centerrod
type=cylinder solid
length=38.1
radius=0.28575
matname=zirc
```

```
shape=upperreflector
type=cylinder solid
length=6.5024
radius=1.82245
matname=graphite
```

```
shape=lowerreflector
type=cylinder solid
length=9.4488
radius=1.82245
matname=graphite
```

```
shape=cladding
type=cylinder shell
length=57.531
radius=1.87706
thickness=0.0508
matname=304LSSC
```

```
shape=fitting
type=cone frustum solid
bottomrad=1.87706
toprad=0.01
altitude=9.05
```

matname=304LSSD

* DOE Canister

shape=outer18pipe
type=cylinder shell
length=299.9
radius=22.86
thickness=0.9525
matname=316LSSE
occurrences=1

shape=dishheadsph
type=sphereseg shell
altitude=5.94
largerad=22.86
smallrad=0.001
thickness=0.9525
matname=316LSSE
occurrences=2

shape=plugplates
type=cylinder solid
length=0.64
radius=3.34
matname=316LSSE
occurrences=2

shape=plugs
type=cylinder solid
length=1.27
radius=1.27
matname=316LSSE
occurrences=2

shape=liftingrings
type=cylinder shell
length=1.27
radius=21.755
thickness=2.5399
matname=316LSSE
occurrences=2

shape=backingring
type=cylinder shell
length=5.08
radius=21.905
thickness=0.47752
matname=316LSSE
occurrences=1

structure=impactplate
component=impactplates1 1
component=impactplates2 1
component=impactplatemeet -1
occurrences=2

shape=impactplates1
type=cylinder solid
length=0.9144
radius=21.275
matname=A516Grade70

shape=impactplates2
type=sphere seg solid
altitude=4.17
largerad=22.19
smallrad=12.04
matname=A516Grade70

shape=impactplatemeet
type=cylinder solid
length=0.001
radius=21.275
matname=A516Grade70

* Basket Structure

shape=sstubes
type=cylinder shell
length=83.6
radius=3.015
thickness=0.55372
matname=316LSSG
occurrences=111

shape=baseplate
type=cylinder solid
length=0.95
radius=21.3
matname=316LSSG
occurrences=3

shape=abstubes
type=cylinder shell
length=83.6
radius=2.46
thickness=0.1
matname=alloy22Gd
occurrences=45

shape=bracket
type=box solid
side1=10.798
side2=0.79
side3=12.928
matname=316LSSG
occurrences=36

* Material Descriptions

material=alloy22Gd

density=8.534171
degrate=2.7043E-12 grams
numelements=10
C=0.012415727
P=0.016554303
S=0.016554303
Si=0.066217212
Cr=17.58894707
Mn=0.413857578
Fe=5.589382029
Ni=45.32982052
Mo=12.4699554
Gd=18.49629586

material=UZrHFLIP
density=5.947
degrate=1.8928E-12 grams
numelements=4
H=1.588616303
Zr=89.86145852
U-238=2.5737
U-235=5.9762

material=zirc
density=6.49
degrate=4.0E-14 grams
numelements=1
Zr=100

material=graphite
density=2.25
degrate=4.0E-14 grams
numelements=1
C=100

material=A516Grade70
density=7.850
degrate=8.7063E-10 grams
numelements=6
C=0.28
Mn=1.045
P=0.035
S=0.035
Si=0.29
Fe=98.315

material=316LSSE
density=7.980
degrate=2.52871E-12 grams
numelements=10
C=0.03
Mn=2.0
P=0.045
S=0.03
Si=1.0
Cr=17.0
Ni=12.0

Mo=2.5
N=0.1
Fe=65.295

material=316LSSG
density=7.980
degrate=2.52871E-12 grams
numelements=10
C=0.03
Mn=2.0
P=0.045
S=0.03
Si=1.0
Cr=17.0
Ni=12.0
Mo=2.5
N=0.1
Fe=65.295

material=304LSSC
density=7.940
degrate=2.51603E-12 grams
numelements=9
C=0.03
Mn=2.0
P=0.045
S=0.03
Si=0.75
Cr=19.0
Ni=10.0
N=0.1
Fe=68.045

material=304LSSD
density=7.940
degrate=2.51603E-12 grams
numelements=9
C=0.03
Mn=2.0
P=0.045
S=0.03
Si=0.75
Cr=19.0
Ni=10.0
N=0.1
Fe=68.045

Sample input file: test.in

```
structure=assembly
component=shape1 10
component=shape2 10
component=shape3 10
component=shape4 10
component=shape5 10
component=shape6 10
component=shape7 10
component=shape8 10
component=shape9 10
component=shape10 10
component=shape11 10
component=shape12 10
component=shape13 10
occurrences=1
```

```
shape=shape1
type=cube solid
side=2.0
matname=316LSS
```

```
shape=shape2
type=cube shell
side=2.0
thickness=0.1
matname=316LSS
```

```
shape=shape3
type=box shell
side1=2.0
side2=3.0
side3=4.0
thickness=0.1
matname=316LSS
```

```
shape=shape4
type=sphere solid
radius=2.0
matname=316LSS
```

```
shape=shape5
type=sphere shell
radius=2.0
thickness=0.1
matname=316LSS
```

```
shape=shape6
type=rtprism solid
side=2.0
length=20.0
matname=316LSS
```


shape=shape7
type=rtprism shell
side=2.0
length=20.0
thickness=0.1
matname=316LSS

shape=shape8
type=etprism solid
side=2.0
length=20.0
matname=316LSS

shape=shape9
type=etprism shell
side=2.0
length=20.0
thickness=0.1
matname=316LSS

shape=shape10
type=hprism solid
side=2.0
length=20.0
matname=316LSS

shape=shape11
type=hprism shell
side=2.0
length=20.0
thickness=0.1
matname=316LSS

shape=shape12
type=helix solid
altitude=25.0
smallradius=0.1
largeradius=0.5
turns=125
matname=316LSS

shape=shape13
type=cone frustrum shell
altitude=5.0
bottomrad=3.0
toprad=2.0
thickness=0.2
matname=316LSS

material=316LSS
density=7.980
degrate=2.516E-12 grams
numelements=10
C=0.03
Mn=2.0
P=0.045
S=0.03

Si=1.0
Cr=17.0
Ni=12.0
Mo=2.5
N=0.1
Fe=65.295

4. Sample output file: triga.out

* DOE Canister
 * Basket Structure
 * Material Descriptions

Composition of alloy22Gd in grams and moles, check vol=5696.601074

Name	Grams	Moles
C	1.242e-002	1.034e-003
P	1.655e-002	5.345e-004
S	1.655e-002	5.163e-004
Si	6.622e-002	2.358e-003
Cr	1.759e+001	3.383e-001
Mn	4.139e-001	7.533e-003
Fe	5.589e+000	1.001e-001
Ni	4.533e+001	7.724e-001
Mo	1.247e+001	1.300e-001
Gd	1.850e+001	1.176e-001

Composition of UZrHFLIP in grams and moles, check vol=42788.152344

Name	Grams	Moles
H	1.589e+000	1.576e+000
Zr	8.986e+001	9.851e-001
U-238	2.574e+000	1.081e-002
U-235	5.976e+000	2.543e-002

Composition of zirc in grams and moles, check vol=1084.851440

Name	Grams	Moles
Zr	1.000e+002	1.096e+000

Composition of graphite in grams and moles, check vol=18474.703125

Name	Grams	Moles
C	1.000e+002	8.326e+000

Composition of A516Grade70 in grams and moles, check vol=11023.248047

Name	Grams	Moles
C	2.800e-001	2.331e-002
Mn	1.045e+000	1.902e-002
P	3.500e-002	1.130e-003
S	3.500e-002	1.091e-003
Si	2.900e-001	1.033e-002
Fe	9.832e+001	1.760e+000

Composition of 316LSSE in grams and moles, check vol=44388.121094

Name	Grams	Moles
C	3.000e-002	2.498e-003
Mn	2.000e+000	3.640e-002
P	4.500e-002	1.453e-003
S	3.000e-002	9.356e-004
Si	1.000e+000	3.561e-002
Cr	1.700e+001	3.269e-001
Ni	1.200e+001	2.045e-001
Mo	2.500e+000	2.606e-002
N	1.000e-001	7.139e-003
Fe	6.529e+001	1.169e+000

Composition of 316LSSG in grams and moles, check vol=96432.812500

Name	Grams	Moles
C	3.000e-002	2.498e-003
Mn	2.000e+000	3.640e-002
P	4.500e-002	1.453e-003
S	3.000e-002	9.356e-004

Si	1.000e+000	3.561e-002
Cr	1.700e+001	3.269e-001
Ni	1.200e+001	2.045e-001
Mo	2.500e+000	2.606e-002
N	1.000e-001	7.139e-003
Fe	6.529e+001	1.169e+000

Composition of 304LSSC in grams and moles, check vol=3774.243164

Name	Grams	Moles
C	3.000e-002	2.498e-003
Mn	2.000e+000	3.640e-002
P	4.500e-002	1.453e-003
S	3.000e-002	9.356e-004
Si	7.500e-001	2.670e-002
Cr	1.900e+001	3.654e-001
Ni	1.000e+001	1.704e-001
N	1.000e-001	7.139e-003
Fe	6.804e+001	1.218e+000

Composition of 304LSSD in grams and moles, check vol=7452.574707

Name	Grams	Moles
C	3.000e-002	2.498e-003
Mn	2.000e+000	3.640e-002
P	4.500e-002	1.453e-003
S	3.000e-002	9.356e-004
Si	7.500e-001	2.670e-002
Cr	1.900e+001	3.654e-001
Ni	1.000e+001	1.704e-001
N	1.000e-001	7.139e-003
Fe	6.805e+001	1.218e+000

BY SHAPE

fuel Area=56863.218750 Volume=42788.152344 Material=UZrHFLIP
 centerrod Area=7649.958984 Volume=1084.851440 Material=zirc
 upperreflector Area=10581.199219 Volume=7531.088867 Material=graphite
 lowerreflector Area=14326.184570 Volume=10943.614258 Material=graphite
 cladding Area=148592.203125 Volume=3774.243164 Material=304LSSC
 fitting Area=14618.901367 Volume=7452.574707 Material=304LSSD
 outer18pipe Area=84356.625000 Volume=40174.828125 Material=316LSSE
 dishheadsph Area=6388.553711 Volume=2994.970459 Material=316LSSE
 plugplates Area=167.047272 Volume=44.859325 Material=316LSSE
 plugs Area=40.536598 Volume=12.870369 Material=316LSSE
 liftingrings Area=653.853699 Volume=830.361572 Material=316LSSE
 backingring Area=1383.111328 Volume=330.231842 Material=316LSSE
 impactplates1 Area=5932.325684 Volume=2600.489990 Material=A516Grade70
 impactplates2 Area=6382.131348 Volume=8425.601563 Material=A516Grade70
 impactplatemeet Area=-5688.128418 Volume=-2.843931 Material=A516Grade70
 sstubes Area=319296.937500 Volume=88400.554688 Material=316LSSG
 baseplate Area=8933.275391 Volume=4062.130859 Material=316LSSG
 abstubes Area=113931.992188 Volume=5696.601074 Material=alloy22Gd
 bracket Area=11400.486328 Volume=3970.125977 Material=316LSSG

BY MATERIAL

alloy22Gd Density=8.53417 Moles=0.118575 Area=27.7883 Volume=5696.6
 UZrHFLIP Density=5.947 Moles=0.620637 Area=13.8691 Volume=42788.2
 zirc Density=6.49 Moles=0.0171724 Area=1.86584 Volume=1084.85
 graphite Density=2.25 Moles=0.101386 Area=6.07497 Volume=18474.7
 A516Grade70 Density=7.85 Moles=0.211055 Area=1.61618 Volume=11023.2
 316LSSE Density=7.98 Moles=0.863944 Area=22.6804 Volume=44388.1

316LSSG Density=7.98 Moles=1.87691 Area=82.8368 Volume=96432.8
304LSSC Density=7.94 Moles=0.0730914 Area=36.242 Volume=3774.24
304LSSD Density=7.94 Moles=0.144325 Area=3.56559 Volume=7452.57

Sample output file: test.out

Composition of 316LSS in grams and moles, check vol=4102.795410

Name	Grams	Moles
C	3.000e-002	2.498e-003
Mn	2.000e+000	3.640e-002
P	4.500e-002	1.453e-003
S	3.000e-002	9.356e-004
Si	1.000e+000	3.561e-002
Cr	1.700e+001	3.269e-001
Ni	1.200e+001	2.045e-001
Mo	2.500e+000	2.606e-002
N	1.000e-001	7.139e-003
Fe	6.529e+001	1.169e+000

BY SHAPE

shape1 Area=240.000000 Volume=80.000000 Material=316LSS
shape2 Area=434.399994 Volume=21.680000 Material=316LSS
shape3 Area=970.400024 Volume=48.480000 Material=316LSS
shape4 Area=502.654846 Volume=335.103210 Material=316LSS
shape5 Area=956.300781 Volume=47.794109 Material=316LSS
shape6 Area=1405.685425 Volume=400.000000 Material=316LSS
shape7 Area=2498.233887 Volume=124.911690 Material=316LSS
shape8 Area=1234.640991 Volume=346.410187 Material=316LSS
shape9 Area=2192.153809 Volume=109.607712 Material=316LSS
shape10 Area=2607.846191 Volume=2078.460938 Material=316LSS
shape11 Area=4661.436035 Volume=233.071808 Material=316LSS
shape12 Area=2472.395996 Volume=123.619804 Material=316LSS
shape13 Area=1536.559082 Volume=153.655930 Material=316LSS

BY MATERIAL

316LSS Density=7.98 Moles=0.0831605 Area=5.51504 Volume=4102.8

File: atwts.in

H	1.00794
He	4.0026
Li	6.941
B	10.811
C	12.0107
N	14.00674
O	15.9994
F	18.9984
Na	22.98977
Mg	24.305
Al	26.98154
Si	28.0855
P	30.97376
S	32.066
Cl	35.4527
K	39.0983
Ca	40.078
Ti	47.867
V	50.9415
Cr	51.9961
Mn	54.93805
Fe	55.845
Co	58.9332
Ni	58.69
Cu	63.546
Zn	65.39
As	74.92160
Sr	87.62
Zr	91.224
Nb	92.90638
Mo	95.94
Tc	98.90628
Ru	101.07
Rh	102.9055
Ag	107.8682
Cd	112.411
In	114.818
Sn	118.71
Cs	132.9054
Ba	137.327
La	138.9055
Ce	140.116
Nd	144.24
Sm	150.36
Eu	151.964
Gd	157.25
Hf	178.49
Ta	180.9479
W	183.84
Pb	207.2
Th	232.0381

U	238.0289
U-233	233.0396
U-234	234.0409
U-235	235.0439
U-236	236.0456
U-238	238.0508
Np	237.048
Pu	239.0521
Er	167.26

ATTACHMENT II. LISTING OF FILES ON COMPACT DISKS

This attachment contains the MS-DOS directory for files placed on the electronic media (Attachment III, two compact disks). The files are of various types:

- 1) Excel files (extension = xls).
- 2) EQ3/6 input files (extensions = 3i or 6i).
- 3) EQ6 output files (text, extension = 6o).
- 4) Tab-delimited text files (extension = txt), with names n?????.elem????.txt. These contain total aqueous moles (*.elem_aqu.txt), total moles in minerals and aqueous phase (*.elem_m_a.txt), total moles in minerals, aqueous phase, and remain special reactants (*.elem_tot.txt), and the total moles in minerals alone (*.elem_min.txt). The *.elem_tot.txt and *.elem_min.txt also have the volume in cm³ of the minerals and total solids (including special reactants) in the system.
- 5) EQ6 text data file used for the calculations, named data0.ymp.
- 6) Selected binary files (bin extension), created by EQ6, used for plotting.
- 7) Figures created by the plotting program (pp.exe, Ref. 30) with files names such as *.wmf and wmf.*.

Below are listed the contents of the DOS directories within the electronic attachment:

The first column is the DOS file name.

The second column lists <DIR> if it is a folder or gives the file size (bytes) if it is a file.

The third and fourth columns are the date and time of the last update.

The fifth column is the file name.

Directory of N-Reactor 2

A516_R~6	XLS	26,624	12-11-00	1:41p	A516_Rate.xls
DATA0	NUC	2,305,224	10-04-99	7:38a	data0.nuc
DATA0	YMP	2,675,993	09-13-00	4:54p	data0.ymp
N-REA~26	DOC	31,232	09-06-00	2:28p	N-reactor_15thEd.doc
N-REA~30	DOC	11,965,440	09-11-00	3:29p	N-Reactor_Tables&Plots.doc
N10#1111	6I	48,815	07-27-00	8:19a	N10#1111.6i
N10#1111	6O	1,804,056	07-27-00	8:27a	n10#1111.6o
N10#1111	6P	48,322	07-27-00	8:27a	n10#1111.6p
N10#1111	BIN	11,257,304	07-27-00	8:27a	N10#1111.bin
N10#~114	TXT	17,500	07-27-00	8:27a	N10#1111.elem_aqu.txt
N10#~116	TXT	16,355	07-27-00	8:27a	N10#1111.elem_min.txt
N10#~118	TXT	16,368	07-27-00	8:27a	N10#1111.elem_tot.txt
N10#1111	TXT	41,784	07-27-00	1:40p	N10#1111.TXT
N10#~122	TXT	34,407	07-27-00	8:43a	N10#1111_min_info.txt
N10#~124	WMF	140,140	07-27-00	11:07a	N10#1111a.wmf
N10#~126	WMF	264,536	07-27-00	1:34p	N10#1111b.wmf
N10&1111	6I	48,889	07-27-00	4:46p	N10&1111.6i
N10&1111	6O	420,946	07-27-00	4:47p	n10&1111.6o
N10&1111	6P	48,224	07-27-00	4:47p	n10&1111.6p
N10&1111	BIN	187,880	07-27-00	4:47p	N10&1111.bin
N10&~138	TXT	4,504	07-27-00	4:47p	N10&1111.elem_aqu.txt
N10&~140	TXT	4,223	07-27-00	4:47p	N10&1111.elem_min.txt
N10&~142	TXT	4,236	07-27-00	4:47p	N10&1111.elem_tot.txt
N10&1111	TXT	663	07-27-00	5:21p	N10&1111.TXT
N10&1111	WMF	270,232	07-27-00	5:19p	N10&1111.wmf
N10A1111	6I	42,436	07-11-00	3:25p	N10A1111.6i
N10A1111	6O	308,743	07-11-00	3:26p	n10a1111.6o
N10A1111	6P	41,587	07-11-00	3:26p	n10a1111.6p
N10A1111	BIN	301,376	07-11-00	3:26p	N10A1111.bin

N10A~160	TXT	3,316	07-11-00	3:26p	N10A1111.elem_aqu.txt
N10A~162	TXT	3,107	07-11-00	3:26p	N10A1111.elem_min.txt
N10A~164	TXT	3,120	07-11-00	3:26p	N10A1111.elem_tot.txt
N10A1201	6I	42,510	08-09-00	2:49p	N10A1201.6i
N10A1201	6O	1,696,437	08-09-00	2:53p	n10a1201.6o
N10A1201	6P	42,235	08-09-00	2:53p	n10a1201.6p
N10A1201	BIN	1,962,072	08-09-00	2:53p	N10A1201.bin
N10A~184	TXT	17,974	08-09-00	2:53p	N10A1201.elem_aqu.txt
N10A~186	TXT	16,757	08-09-00	2:53p	N10A1201.elem_min.txt
N10A~188	TXT	16,770	08-09-00	2:53p	N10A1201.elem_tot.txt
N10A~190	XLS	56,832	07-25-00	8:21a	N10A1201.elem_tot.xls
N10A1201	TX0	18,279	07-19-00	2:17p	N10A1201.TX0
N10A1201	TXT	21,839	07-18-00	4:50p	N10A1201.TXT
N10A~198	TXT	44,365	08-09-00	1:54p	N10A1201_min_info.txt
N10A~200	WMF	132,062	07-19-00	1:16p	N10A1201a.wmf
N10A~202	WMF	130,472	07-19-00	1:27p	N10A1201b.wmf
N10A~204	WMF	139,284	07-24-00	6:31p	N10A1201c.wmf
N10B1011	6I	45,125	08-09-00	3:10p	n10b1011.6i
N10B1011	6O	1,294,074	08-09-00	3:15p	n10b1011.6o
N10B1011	6P	44,724	08-09-00	3:15p	n10b1011.6p
N10B1011	BIN	9,715,888	08-09-00	3:14p	N10B1011.bin
N10B~246	TXT	12,085	08-09-00	3:14p	N10B1011.elem_aqu.txt
N10B~248	TXT	11,300	08-09-00	3:14p	N10B1011.elem_min.txt
N10B~250	TXT	11,313	08-09-00	3:14p	N10B1011.elem_tot.txt
N10B1011	TXT	89,088	07-20-00	6:50p	N10B1011.TXT
N10B1011	TXU	124,229	07-20-00	6:51p	N10B1011.TXU
N10B~256	WMF	204,640	07-20-00	6:08p	N10B1011a.wmf
N10B~258	WMF	134,240	07-20-00	6:22p	N10B1011b.wmf
N10G2111	6I	49,333	08-10-00	11:09a	N10g2111.6i
N10G2111	6O	2,407,190	08-10-00	11:18a	n10g2111.6o
N10G2111	6P	49,390	08-10-00	11:18a	n10g2111.6p
N10G2111	BIN	11,837,768	08-10-00	11:18a	N10G2111.bin
N10G~312	TXT	23,637	08-10-00	11:18a	N10G2111.elem_aqu.txt
N10G~314	TXT	22,084	08-10-00	11:18a	N10G2111.elem_min.txt
N10G~316	TXT	22,097	08-10-00	11:18a	N10G2111.elem_tot.txt
N10G~318	TXT	38,279	08-10-00	11:05a	N10g2111_min_info.txt
N10H1111	6I	48,741	07-19-00	5:19p	N10h1111.6i
N10H1111	6O	1,864,458	07-19-00	5:32p	n10h1111.6o
N10H1111	6P	48,174	07-19-00	5:32p	n10h1111.6p
N10H1111	BIN	11,343,536	07-19-00	5:31p	N10H1111.bin
N10H~366	TXT	18,222	07-19-00	5:31p	N10H1111.elem_aqu.txt
N10H~368	TXT	17,029	07-19-00	5:31p	N10H1111.elem_min.txt
N10H~370	TXT	17,042	07-19-00	5:31p	N10H1111.elem_tot.txt
N10H1111	TXT	163,780	07-24-00	1:23p	N10H1111.TXT
N10H~374	TXT	35,399	07-19-00	5:33p	N10h1111_min_info.txt
N10H~378	WMF	260,586	07-19-00	6:19p	N10h1111a.wmf
N10H~380	WMF	126,900	07-19-00	6:34p	N10h1111b.wmf
N10H1112	6I	48,963	07-26-00	8:15a	N10h1112.6i
N10H1112	6O	7,473,929	07-26-00	3:39p	n10h1112.6o
N10H1112	6P	48,372	07-26-00	3:39p	n10h1112.6p
N10H1112	BIN	0	07-26-00	3:39p	N10H1112.bin
N10H~412	TXT	63,347	07-26-00	3:39p	N10H1112.elem_aqu.txt
N10H~414	TXT	59,154	07-26-00	2:39p	N10H1112.elem_min.txt
N10H~416	TXT	59,167	07-26-00	2:39p	N10H1112.elem_tot.txt
N10H1112	TXT	303,549	07-26-00	4:33p	N10H1112.TXT
N10H~422	TXT	116,595	07-26-00	2:43p	N10h1112_min_info.txt
N10H~424	WMF	149,164	07-26-00	5:04p	N10h1112a.wmf

N10H~426	WMF	136,302	07-26-00	5:16p	N10h1112b.wmf
N10H1211	6I	49,141	08-03-00	9:31a	N10h1211.6i
N10H1211	6O	2,566,252	08-03-00	2:39p	n10h1211.6o
N10H1211	6P	49,196	08-03-00	2:39p	n10h1211.6p
N10H1211	BIN	12,468,736	08-03-00	2:39p	N10h1211.bin
N10H~482	TXT	24,806	08-03-00	2:39p	N10h1211.elem_aqu.txt
N10H~484	TXT	23,277	08-03-00	2:39p	N10h1211.elem_min.txt
N10H~486	TXT	23,290	08-03-00	2:39p	N10h1211.elem_tot.txt
N10H1211	TXT	43,319	08-03-00	3:54p	N10H1211.TXT
N10H~490	TXT	41,156	08-03-00	2:41p	N10h1211_min_info.txt
N10H2111	6I	48,815	07-24-00	2:15p	N10h2111.6i
N10H2111	6O	2,691,047	07-24-00	3:27p	n10h2111.6o
N10H2111	6P	48,528	08-09-00	12:25p	n10h2111.6p
N10H2111	BIN	4,403,776	08-09-00	12:25p	N10h2111.bin
N10H~520	TXT	18,944	08-09-00	12:25p	N10h2111.elem_aqu.txt
N10H~522	TXT	17,703	08-09-00	12:25p	N10h2111.elem_min.txt
N10H~524	TXT	17,716	08-09-00	12:25p	N10h2111.elem_tot.txt
N10H2111	TXT	384,883	07-24-00	5:11p	N10H2111.TXT
N10H~530	TXT	59,043	07-24-00	3:06p	N10h2111_min_info.txt
N10H~532	WMF	183,422	07-25-00	10:27a	N10h2111a.wmf
N10H~534	WMF	222,866	07-25-00	9:42a	N10h2111b.wmf
N10H~538	WMF	113,550	07-25-00	10:12a	N10h2111c.wmf
N11&1111	6I	47,007	07-27-00	4:56p	n11&1111.6i
N11&1111	6O	1,446,456	07-27-00	5:01p	n11&1111.6o
N11&1111	6P	47,409	07-27-00	5:01p	n11&1111.6p
N11&1111	BIN	10,179,920	07-27-00	5:01p	N11&1111.bin
N11&~582	TXT	13,890	07-27-00	5:01p	N11&1111.elem_aqu.txt
N11&~584	TXT	12,985	07-27-00	5:01p	N11&1111.elem_min.txt
N11&~586	TXT	12,998	07-27-00	5:01p	N11&1111.elem_tot.txt
N11&1111	TXT	38,139	07-27-00	5:21p	N11&1111.TXT
N11A1111	6I	40,444	07-11-00	4:25p	n11a1111.6i
N11A1111	6O	1,443,415	07-17-00	10:01a	n11a1111.6o
N11A1111	6P	41,166	07-17-00	10:01a	n11a1111.6p
N11A1111	BIN	9,400,856	07-17-00	10:01a	N11A1111.bin
N11A~630	TXT	14,484	07-17-00	10:01a	N11A1111.elem_aqu.txt
N11A~632	TXT	13,507	07-17-00	10:01a	N11A1111.elem_min.txt
N11A~634	TXT	13,520	07-17-00	10:01a	N11A1111.elem_tot.txt
N11A1211	6I	41,593	07-11-00	2:52p	n11a1211.6i
N11H1112	6I	39,713	07-26-00	2:07p	n11h1112.6i
N11H1112	6O	161,000	07-26-00	2:44p	n11h1112.6o
N11H1112	6P	40,017	07-26-00	2:44p	n11h1112.6p
N11H1112	BIN	234,752	07-26-00	2:44p	N11H1112.bin
N11H~648	TXT	1,616	07-26-00	2:44p	N11H1112.elem_aqu.txt
N11H~650	TXT	1,527	07-26-00	2:44p	N11H1112.elem_min.txt
N11H~652	TXT	1,540	07-26-00	2:44p	N11H1112.elem_tot.txt
N11H~654	TXT	4,272	07-26-00	2:45p	N11h1112_min_info.txt
N11H2111	6I	38,270	08-09-00	12:37p	n11h2111.6i
N11H2111	6O	997,595	08-09-00	11:43a	n11h2111.6o
N11H2111	6P	38,348	08-09-00	11:43a	n11h2111.6p
N11H2111	BIN	7,289,904	08-09-00	11:43a	N11h2111.bin
N11H~688	TXT	8,603	08-09-00	11:43a	N11h2111.elem_aqu.txt
N11H~690	TXT	8,010	08-09-00	11:43a	N11h2111.elem_min.txt
N11H~692	TXT	8,023	08-09-00	11:43a	N11h2111.elem_tot.txt
N11H~694	TXT	11,282	08-08-00	12:17p	N11h2111_min_info.txt
N12H1112	6I	40,091	07-26-00	2:09p	n12h1112.6i
N12H1112	6O	396,602	07-26-00	2:11p	n12h1112.6o
N12H1112	6P	40,165	07-26-00	2:11p	n12h1112.6p

N12H1112	BIN	2,483,104	07-26-00	2:11p	N12H1112.bin
N12H~714	TXT	3,782	07-26-00	2:11p	N12H1112.elem_aqu.txt
N12H~716	TXT	3,549	07-26-00	2:11p	N12H1112.elem_min.txt
N12H~718	TXT	3,562	07-26-00	2:11p	N12H1112.elem_tot.txt
N12H1112	TXT	9,357	07-26-00	4:34p	N12H1112.TXT
N12H1112	WMF	160,764	07-26-00	5:24p	N12h1112.wmf
N12H~724	TXT	7,660	07-26-00	2:15p	N12h1112_min_info.txt
N12H2111	6I	39,404	08-08-00	10:37a	n12h2111.6i
N12H2111	6O	462,165	08-08-00	10:38a	n12h2111.6o
N12H2111	6P	39,552	08-08-00	10:38a	n12h2111.6p
N12H2111	BIN	1,445,192	08-08-00	10:38a	N12h2111.bin
N12H~740	TXT	5,226	08-08-00	10:38a	N12h2111.elem_aqu.txt
N12H~742	TXT	4,897	08-08-00	10:38a	N12h2111.elem_min.txt
N12H~744	TXT	4,910	08-08-00	10:38a	N12h2111.elem_tot.txt
N13H2111	6I	39,626	08-08-00	10:42a	n13h2111.6i
N13H2111	6O	1,055,260	08-08-00	10:47a	n13h2111.6o
N13H2111	6P	39,700	08-08-00	10:47a	n13h2111.6p
N13H2111	BIN	7,010,160	08-08-00	11:47a	N13h2111.bin
N13H~778	TXT	11,363	08-08-00	11:47a	N13h2111.elem_aqu.txt
N13H~780	TXT	10,626	08-08-00	10:47a	N13h2111.elem_min.txt
N13H~782	TXT	10,639	08-08-00	10:47a	N13h2111.elem_tot.txt
N40H1111	6I	47,617	07-31-00	12:20p	N40h1111.6i
N40H1111	6O	2,652,944	07-31-00	12:55p	n40h1111.6o
N40H1111	6P	46,977	07-31-00	12:55p	n40h1111.6p
N40H1111	BIN	22,624,680	07-31-00	12:55p	N40h1111.bin
N40H~866	TXT	24,720	07-31-00	12:55p	N40h1111.elem_aqu.txt
N40H~868	TXT	23,095	07-31-00	12:55p	N40h1111.elem_min.txt
N40H~870	TXT	23,108	07-31-00	12:55p	N40h1111.elem_tot.txt
N40H1111	TXT	84,174	07-31-00	1:30p	N40H1111.TXT
N40H~874	TXT	45,575	07-31-00	1:19p	N40h1111_min_info.txt
N40H1112	6I	47,913	07-31-00	5:53p	N40h1112.6i
N40H1112	6O	7,494,087	08-01-00	9:32a	n40h1112.6o
N40H1112	6P	47,323	08-01-00	9:32a	n40h1112.6p
N40H1112	BIN	81,716,160	08-01-00	10:32a	N40h1112.bin
N40~1146	TXT	64,069	08-01-00	9:32a	N40h1112.elem_aqu.txt
N40~1148	TXT	59,828	08-01-00	9:32a	N40h1112.elem_min.txt
N40~1150	TXT	59,841	08-01-00	9:32a	N40h1112.elem_tot.txt
N40H1112	TXT	30,440	08-01-00	9:59a	N40H1112.TXT
N40~1154	TXT	121,789	08-01-00	9:34a	N40h1112_min_info.txt
N40H2111	6I	47,691	07-31-00	12:21p	N40h2111.6i
N40H2111	6O	3,359,190	08-03-00	8:45a	n40h2111.6o
N40H2111	6P	47,371	08-03-00	8:45a	n40h2111.6p
N40H2111	BIN	23,903,840	08-03-00	8:45a	N40h2111.bin
N40~1244	TXT	31,940	08-03-00	8:45a	N40h2111.elem_aqu.txt
N40~1246	TXT	29,835	08-03-00	8:45a	N40h2111.elem_min.txt
N40~1248	TXT	29,848	08-03-00	8:45a	N40h2111.elem_tot.txt
N40H2111	TXT	88,521	07-31-00	1:49p	N40H2111.TXT
N40~1252	TXT	66,595	07-31-00	12:46p	N40h2111_min_info.txt
N41H1112	6I	39,565	08-01-00	8:56a	n41h1112.6i
N41H1112	6O	160,702	08-01-00	8:57a	n41h1112.6o
N41H1112	6P	39,869	08-01-00	8:57a	n41h1112.6p
N41H1112	BIN	234,752	08-01-00	8:57a	N41h1112.bin
N41~1264	TXT	1,616	08-01-00	8:57a	N41h1112.elem_aqu.txt
N41~1266	TXT	1,527	08-01-00	8:57a	N41h1112.elem_min.txt
N41~1268	TXT	1,540	08-01-00	8:57a	N41h1112.elem_tot.txt
N41~1270	TXT	4,272	08-01-00	8:58a	N41h1112_min_info.txt
N42H1112	6I	39,943	08-01-00	9:02a	n42h1112.6i

N42H1112	6O	448,559	08-01-00	9:03a	n42h1112.6o
N42H1112	6P	40,017	08-01-00	9:03a	n42h1112.6p
N42H1112	BIN	2,483,104	08-01-00	9:03a	N42h1112.bin
N42~1288	TXT	4,504	08-01-00	9:03a	N42h1112.elem_aqu.txt
N42~1290	TXT	4,223	08-01-00	9:03a	N42h1112.elem_min.txt
N42~1292	TXT	4,236	08-01-00	9:03a	N42h1112.elem_tot.txt
N42H1112	TXT	9,353	08-01-00	10:05a	N42H1112.TXT
N42~1296	TXT	8,724	08-01-00	9:04a	n42h1112_min_info.txt
NREACTOR	XLS	143,872	01-16-01	11:04a	NReactor.xls
NRE~1304	XLS	4,894,208	12-18-00	4:50p	NReactor_U.xls
VOLAREA		<DIR>	01-16-01	11:21a	volarea
WMF	022	345,912	07-28-00	1:48p	WMF.022
WMF	023	346,812	07-28-00	1:49p	WMF.023
WMF	024	108,966	07-28-00	1:56p	WMF.024
WMF	025	343,092	07-28-00	2:03p	WMF.025
WMF	029	383,626	07-31-00	1:41p	WMF.029
WMF	030	141,032	07-31-00	1:46p	WMF.030
WMF	031	328,016	07-31-00	1:58p	WMF.031
WMF	032	241,960	07-31-00	2:10p	WMF.032
WMF	033	118,088	07-31-00	2:15p	WMF.033
WMF	034	224,124	08-01-00	9:53a	WMF.034
WMF	035	117,930	08-01-00	9:58a	WMF.035
WMF	036	158,884	08-01-00	10:05a	WMF.036
WMF	037	332,270	08-03-00	3:43p	WMF.037
WMF	038	161,906	08-03-00	3:53p	WMF.038

224 file(s) 308,519,821 bytes

Directory of D:\volarea

.		<DIR>	08-03-00	3:53p	.
..		<DIR>	08-03-00	3:53p	..
ATWTS	IN	1,020	11-29-00	1:19p	atwts.in
DATAMIA	IN	3,045	07-06-00	10:22a	dataMIA.in
DATAMIA	OUT	4,337	07-06-00	10:24a	dataMIA.out
DATAMIV	IN	2,885	07-06-00	10:43a	dataMIV.in
DATAMIV	OUT	3,918	07-06-00	10:44a	dataMIV.out
RUNVOL	BAT	11	11-29-00	11:29a	runvol.bat
TEST	IN	1,565	11-29-00	11:27a	test.in
TEST	OUT	1,434	11-29-00	1:39p	test.out
TEST	TXT	1,565	11-29-00	11:27a	test.txt
TRIGA	IN	3,978	11-29-00	11:30a	triga.in
TRIGA	OUT	5,990	11-29-00	11:30a	Triga.out
TRIGA	TXT	3,978	11-29-00	11:27a	Triga.txt
VOLAR~30	XLS	30,208	11-29-00	2:16p	volarea_qualify.xls
VOLAREA	C	34,054	05-18-00	11:05a	volarea.c
VOLAREA	EXE	229,428	05-18-00	11:04a	volarea.exe

15 file(s) 327,416 bytes

Total files listed:

239 file(s) 308,847,237 bytes